



LABORATÓRIO DE INSTRUMENTAÇÃO  
E FÍSICA EXPERIMENTAL DE PARTÍCULAS  
*partículas e tecnologia*

# [ MEASURING THE NEUTRON CAPTURE CROSS SECTION OF ARGON ]

— *A Feasibility Study*



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# Carolina Machado

Project Supervisors: Sofia Andringa and Valentina Lozza



## MOTIVATION

- LAr is currently used as a detector material for several low energy and rare event searches which neutrons might interfere with and its' cross section is poorly known.

## OBJECTIVES

- Identifying the best describing theoretical models for the experimental data and the energy ranges that show the largest discrepancies between data and theoretical models and that require further studies.

# CROSS SECTION, $\sigma$

Measure of the probability that a specific process will take place when some kind of radiant excitation intersects a localized phenomenon.

## **ELASTIC CROSS SECTION, $\sigma_E$**

Probability of a scattering process occurring when two particles elastically collide only changing its directions, where the kinetic energy and momenta are conserved.

## **NON-ELASTIC CROSS SECTION, $\sigma_{NE}$**

= Total Cross-Section – Elastic Cross-Section

Probability of a scattering process occurring when two particles collide changing the target nucleus, and creating new particles, easily detectable.

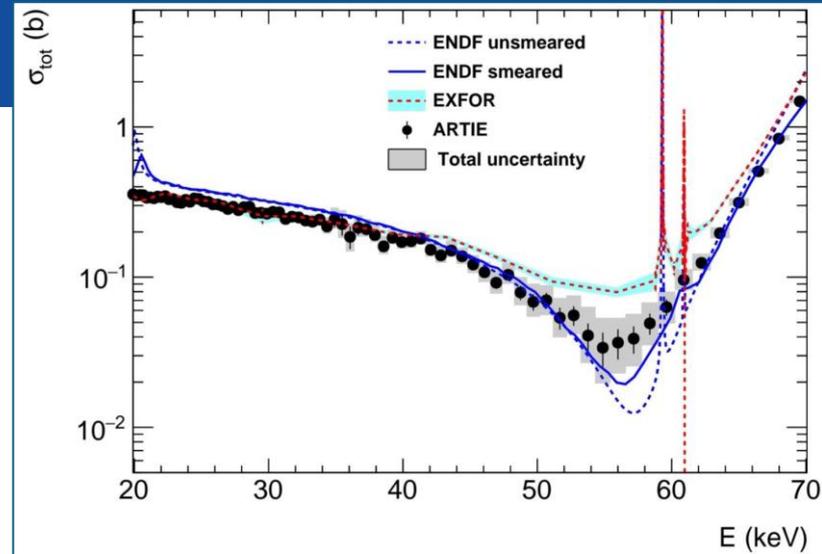
# MEAN FREE PATH

$$\lambda[cm] = \frac{1}{\sigma[cm^2]n[cm^{-3}]}$$

n represents the density of particles in the medium.

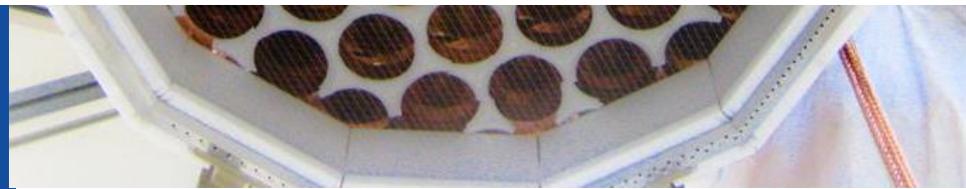
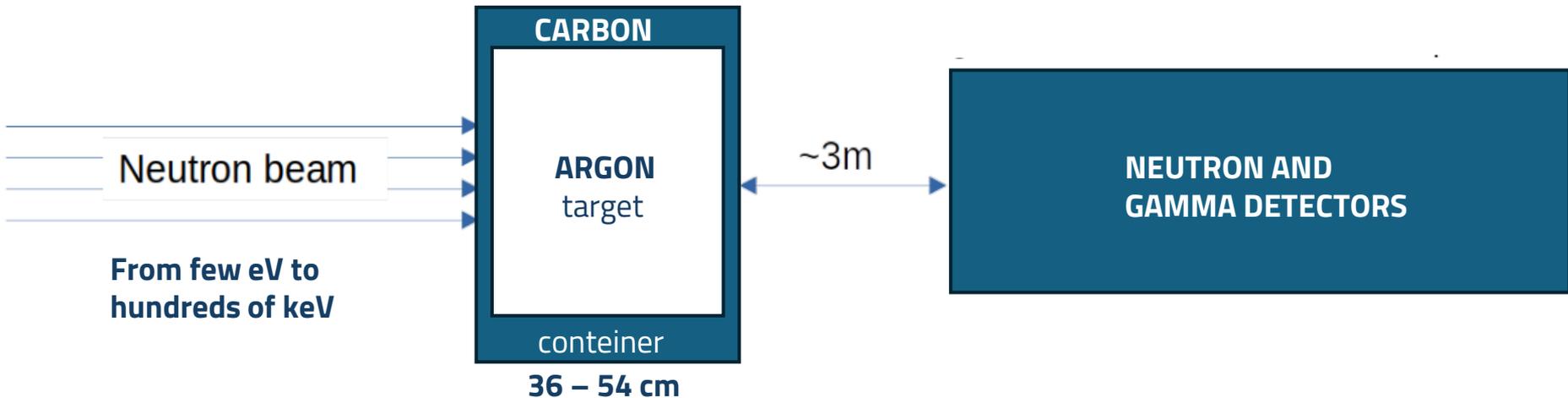
Particles' Path before an interaction.

It was recently verified that the neutron's free path is approximately 10 times bigger that it was known before, due to the Neutron-Argon Cross Section being 10 times smaller. These new measurements are more in agreement with the theoretical models.



Neutron-Argon total cross section as a function of energy

# MAREx at CERN Experimental Setup



# Carbon and Argon

## Natural Abundances

### Argon 40

99,6%

18 protons  
22 neutrons

### Argon 36

0,336%

18 protons  
18 neutrons

### Argon 38

0,063%

18 protons  
20 neutrons

### Carbon 12

98,89%

6 protons  
6 neutrons

### Carbon 13

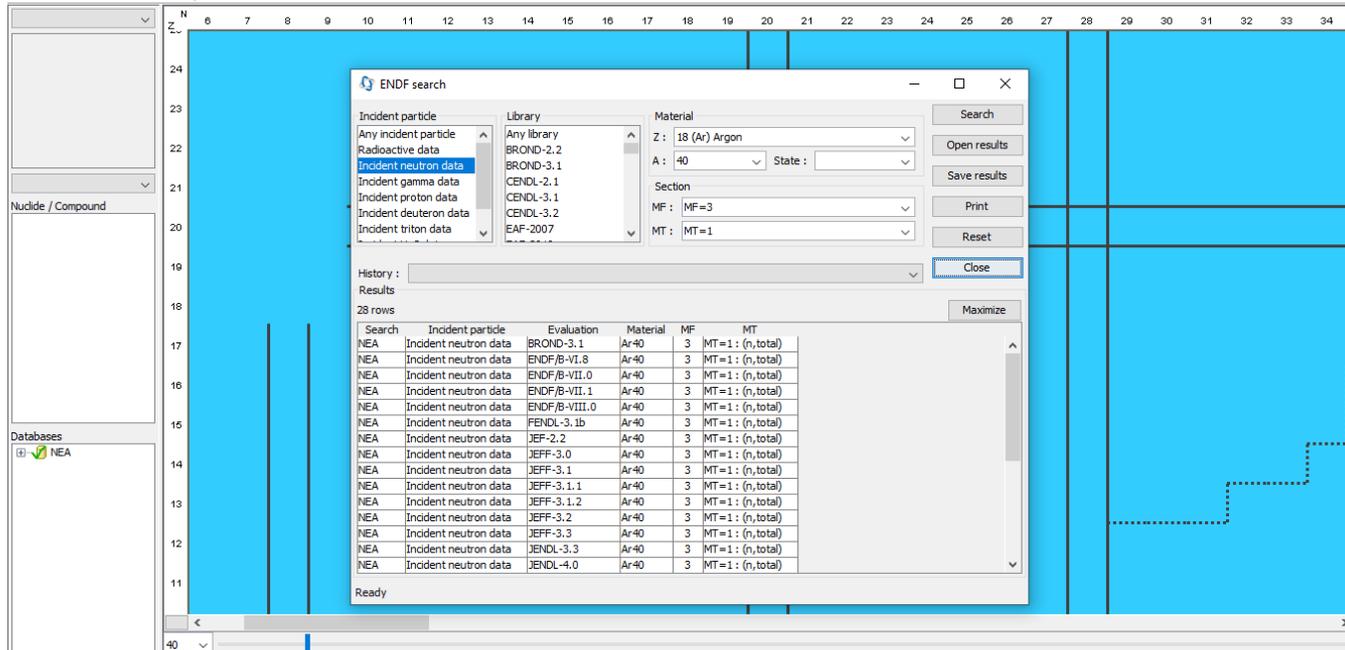
1,11%

6 protons  
7 neutrons

# Janis Web 4.1

 <https://www.oecd-nea.org/janisweb/>

## ENDF Theoretical Models: Evaluated Nuclear Data Files EXFOR Experimental Data: Experimental Nuclear Reaction Data



History : Results  
28 rows

Search	Incident particle	Evaluation	Material	MF	MT
NEA	Incident neutron data	BROND-3.1	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	ENDF/B-VI.8	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	ENDF/B-VII.0	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	ENDF/B-VII.1	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	ENDF/B-VIII.0	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	FENDL-3.1b	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEF-2.2	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.0	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.1.1	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.1.1.1	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.1.1.2	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.2	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JEFF-3.3	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JENDL-3.3	Ar-40	3	MT=1 : (n,total)
NEA	Incident neutron data	JENDL-4.0	Ar-40	3	MT=1 : (n,total)

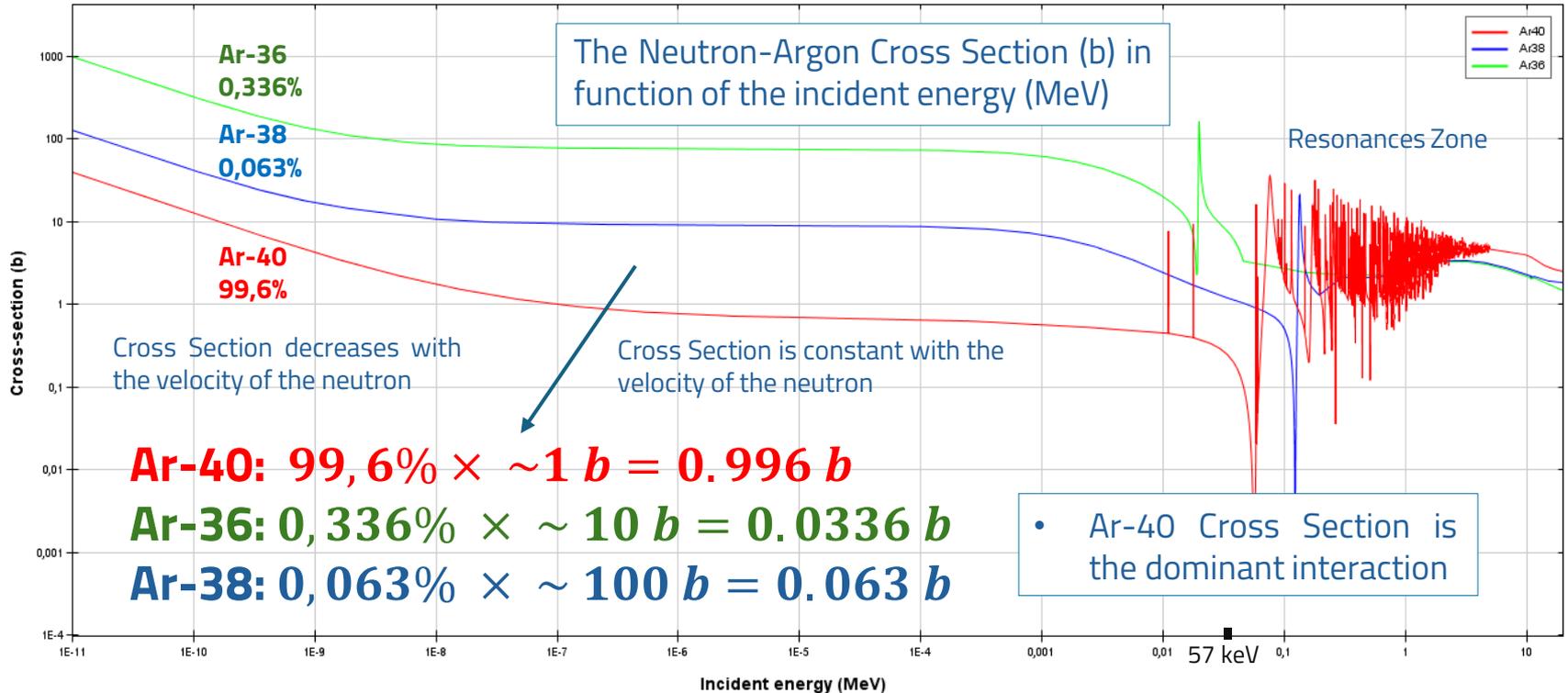
Janis Web 4.1 allows to establish a comparison between Neutron-Argon and Neutron-Carbon Cross Sections

1.

**Argon (target)**

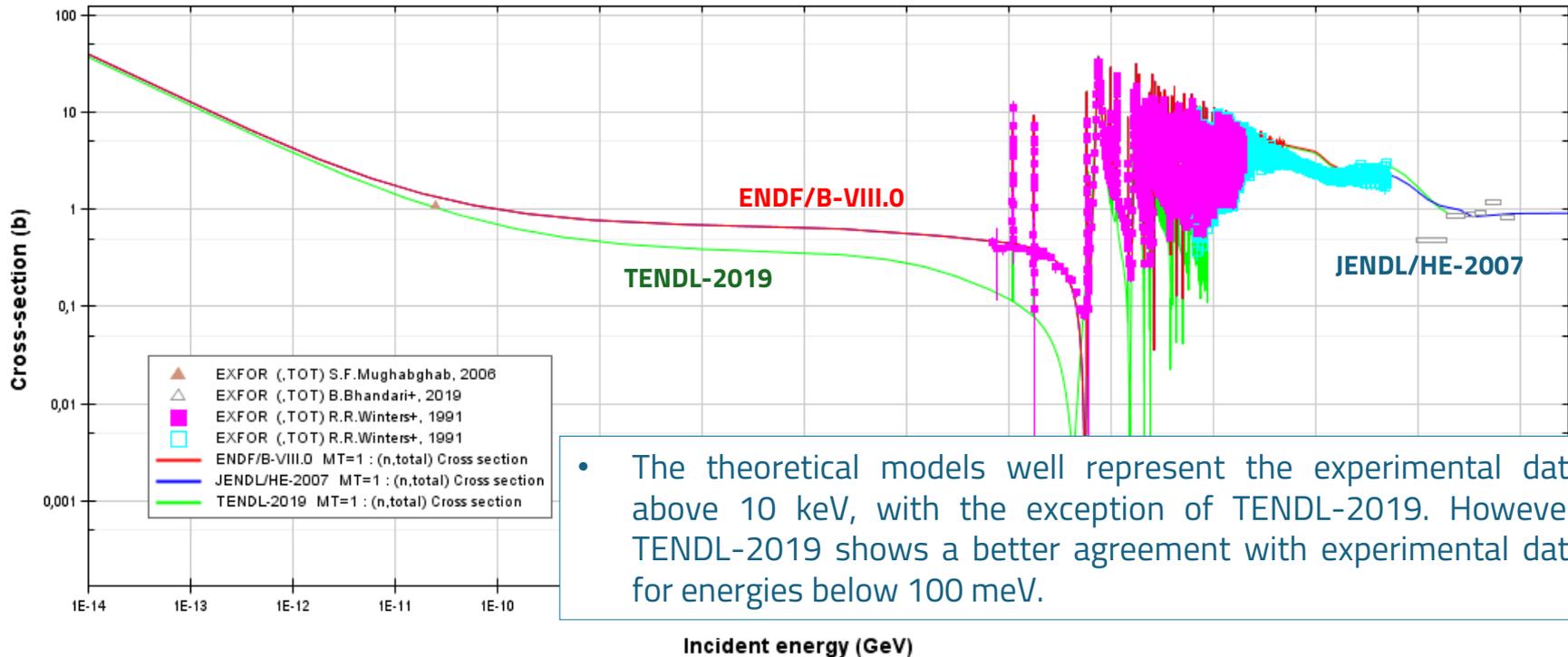
# Total $^{40}\text{Ar}$ , $^{36}\text{Ar}$ and $^{38}\text{Ar}$ Cross Section

Incident neutron data / ENDF/B-VIII.0 // MT=1 : (n,total) / Cross section



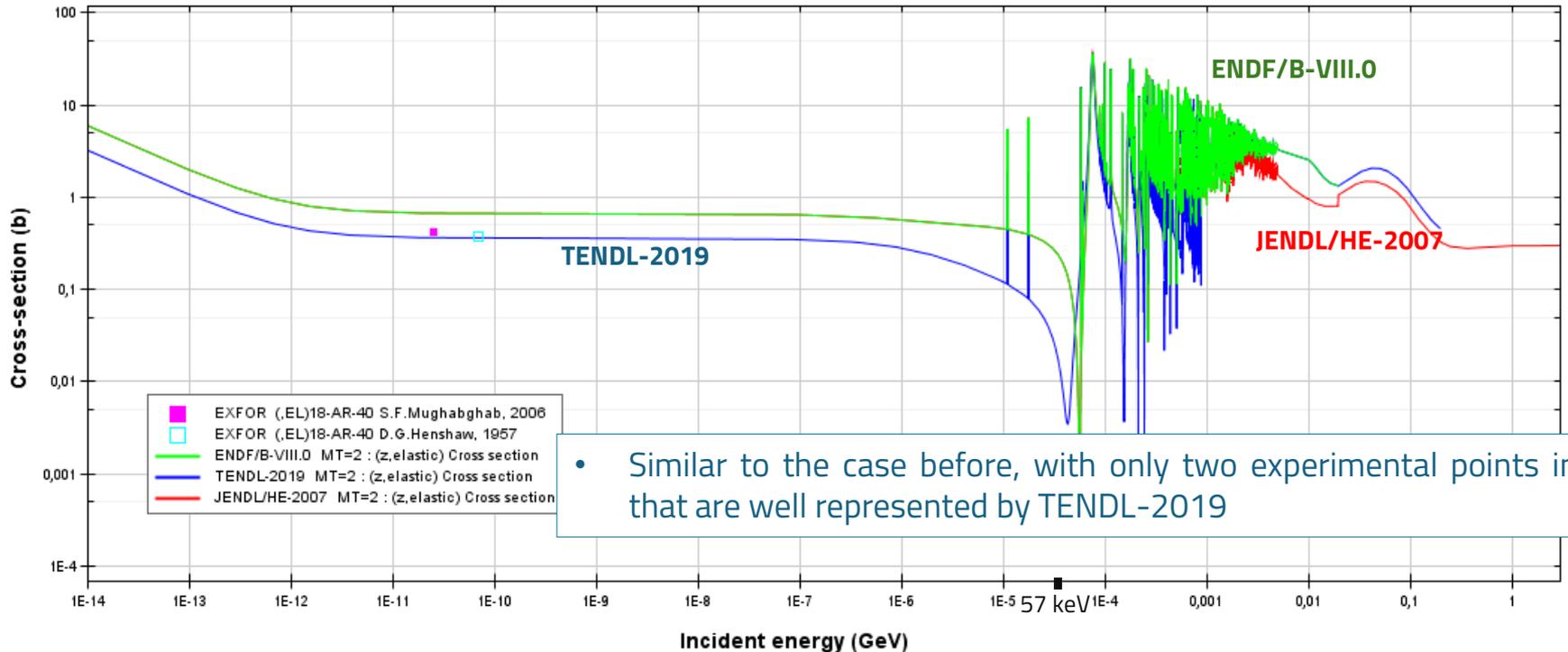
# Total $^{40}\text{Ar}$ Cross Section

Incident neutron data // Ar40 //



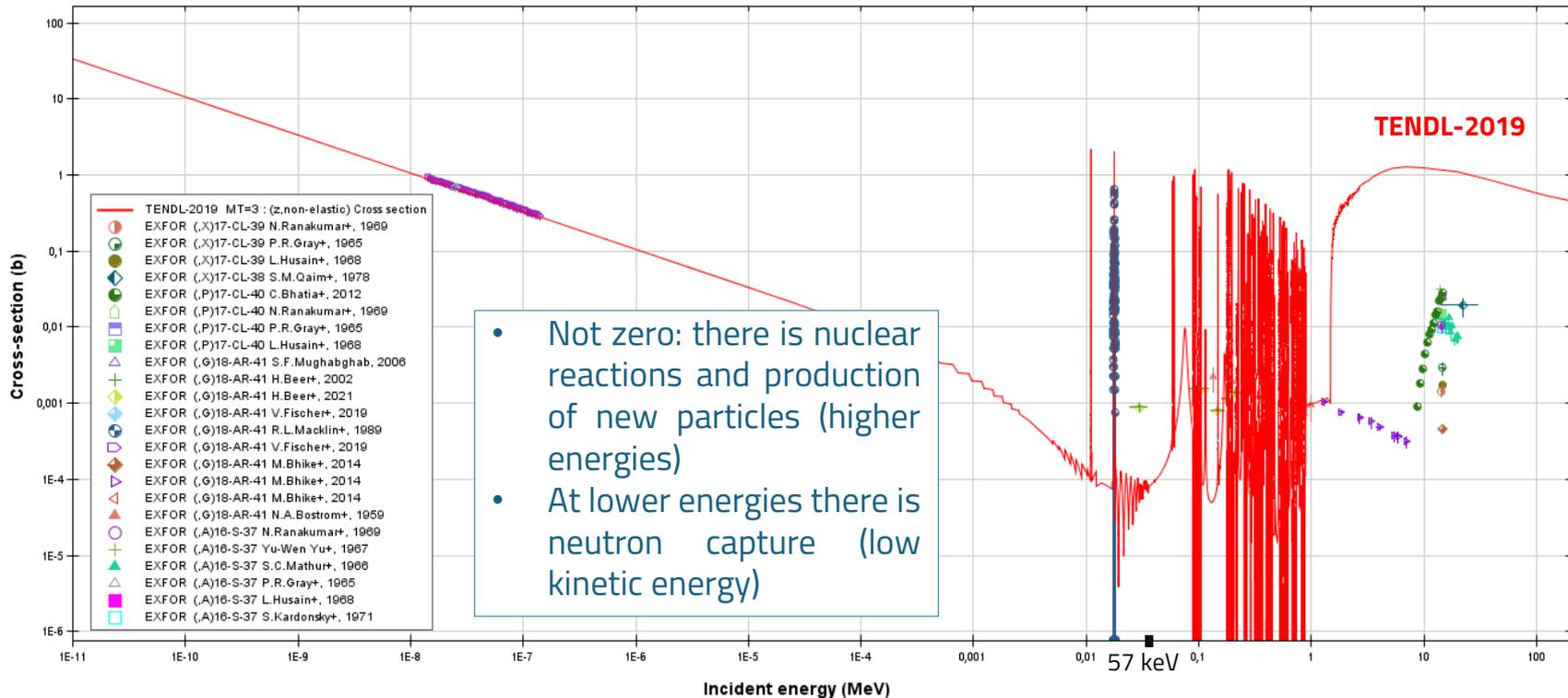
# Elastic $^{40}\text{Ar}$ Cross Section

Incident neutron data // Ar40 //



# Non-Elastic $^{40}\text{Ar}$ Cross Section

Incident neutron data // Ar40 //

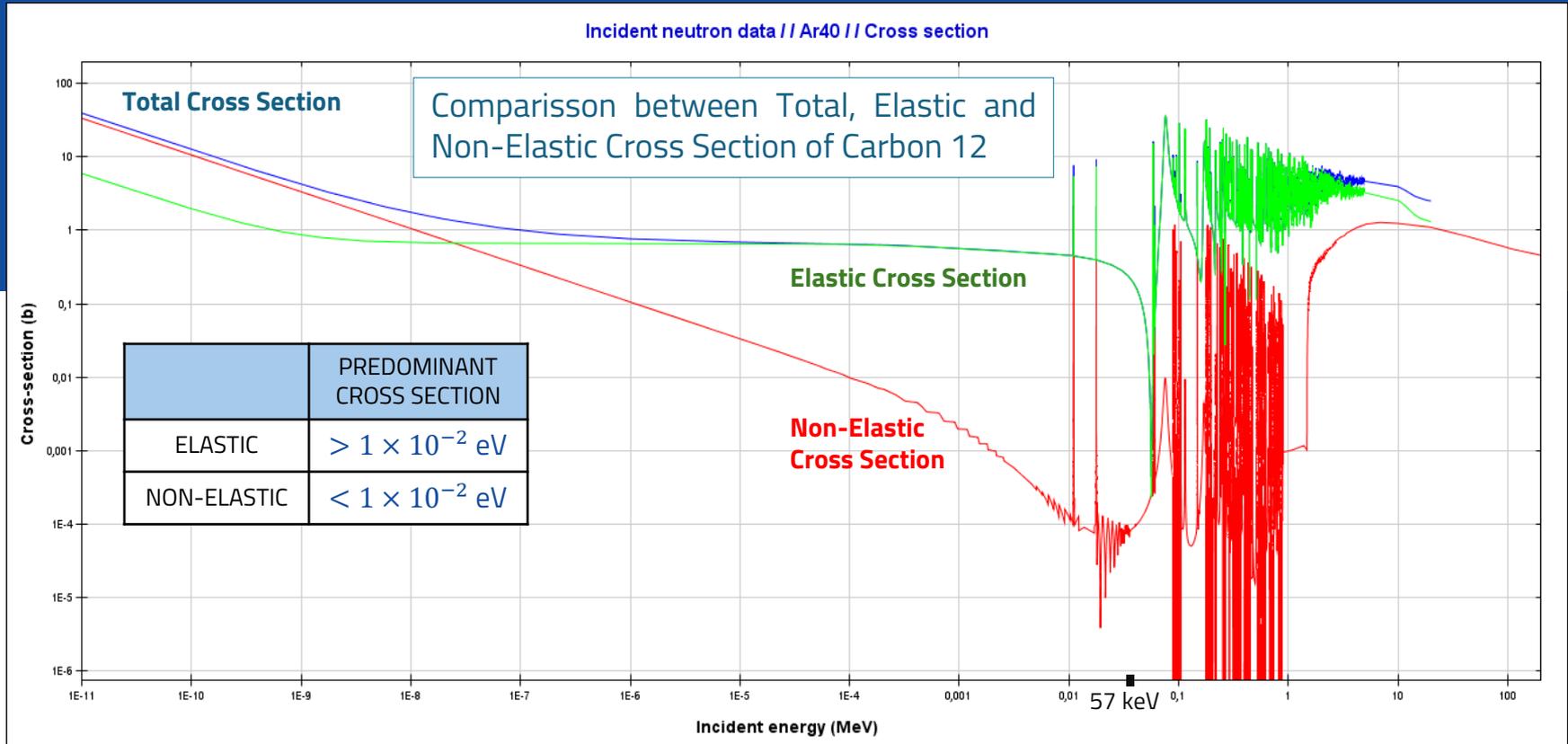


## DETECTED NUCLEAR REACTIONS

ENERGY	EMISSION
[10; 100] MeV	$n + \text{Ar-40} \rightarrow \alpha + \text{S-37}$
[0.01; 0.1] eV / [1; 10] MeV	$n + \text{Ar-40} \rightarrow \gamma + \text{Ar-41}$ (n-capture)
[8; 20] MeV	$n + \text{Ar-40} \rightarrow p + \text{Cl-40}$
[0.01; 0.05] MeV	$n + \text{Ar-40} \rightarrow X(2n+p) + \text{Cl-38}$ $n + \text{Ar-40} \rightarrow X(n+p) + \text{Cl-39}$

**TERMAL NEUTRONS:** Specific energy state of neutrons are characterized by their kinetic energy, which is roughly equivalent to the thermal energy of their surrounding environment, typically at room temperature and are more likely to interact with atomic nuclei due to their lower kinetic energy.

# Total, Elastic and Non-Elastic $^{40}\text{Ar}$ Cross Section

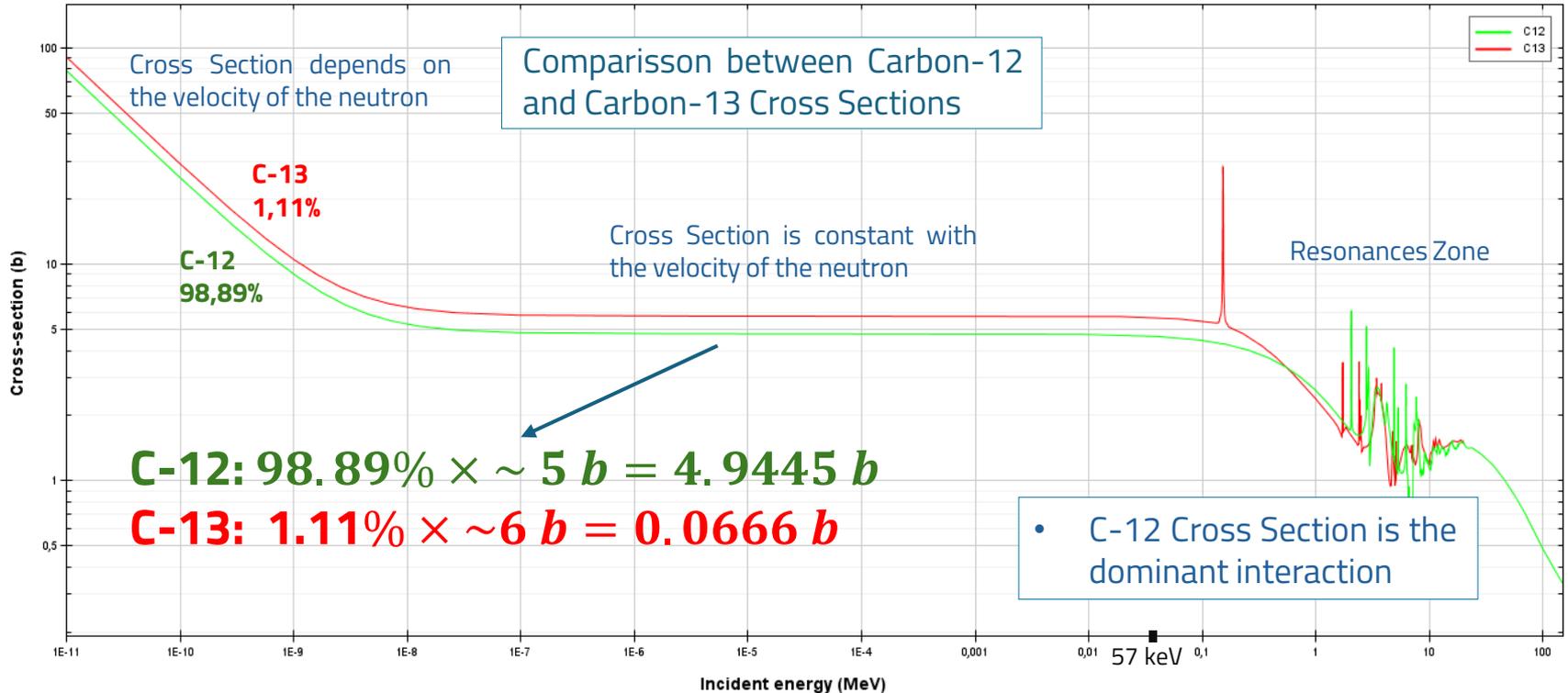


2.

**Carbon (container)**

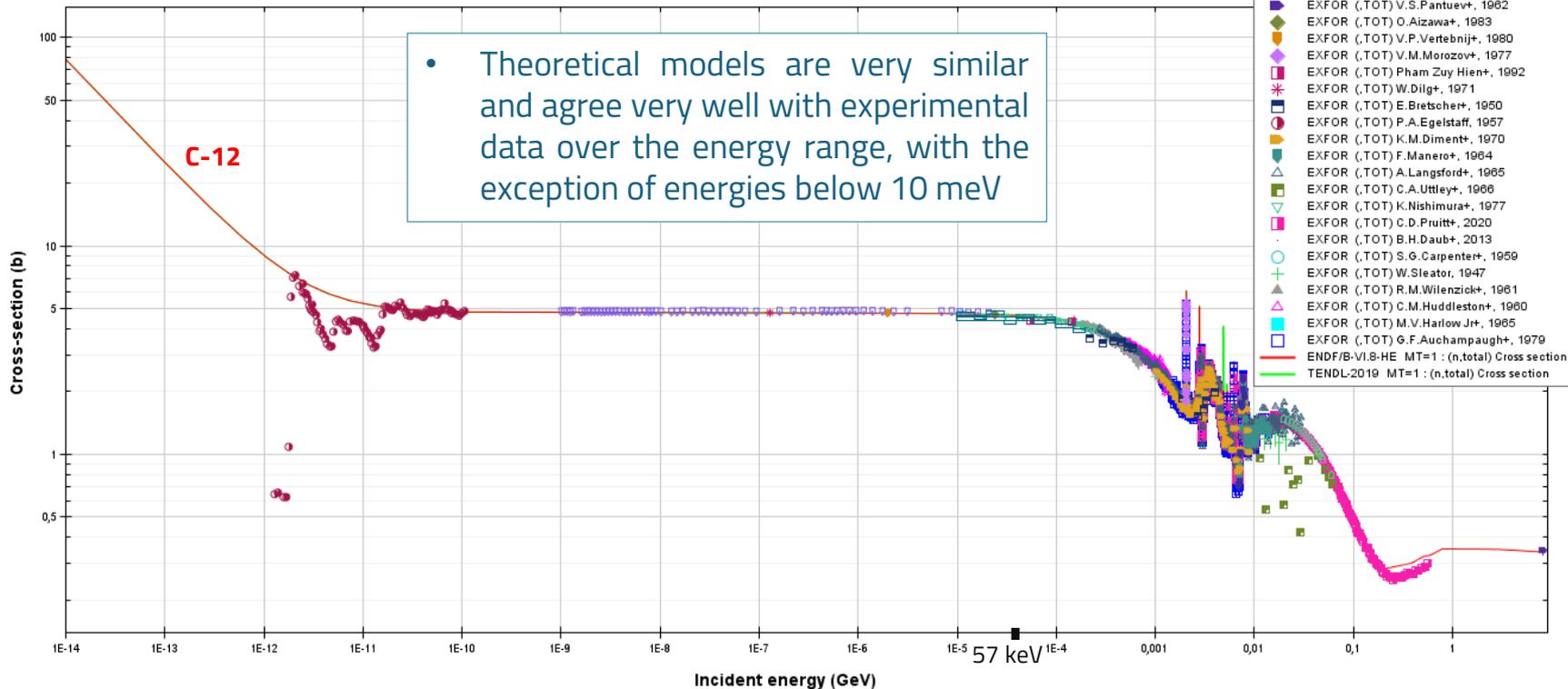
# Total 12C and 13C Cross Section

Incident neutron data / TENDL-2019 // MT=1 : (n,total) / Cross section

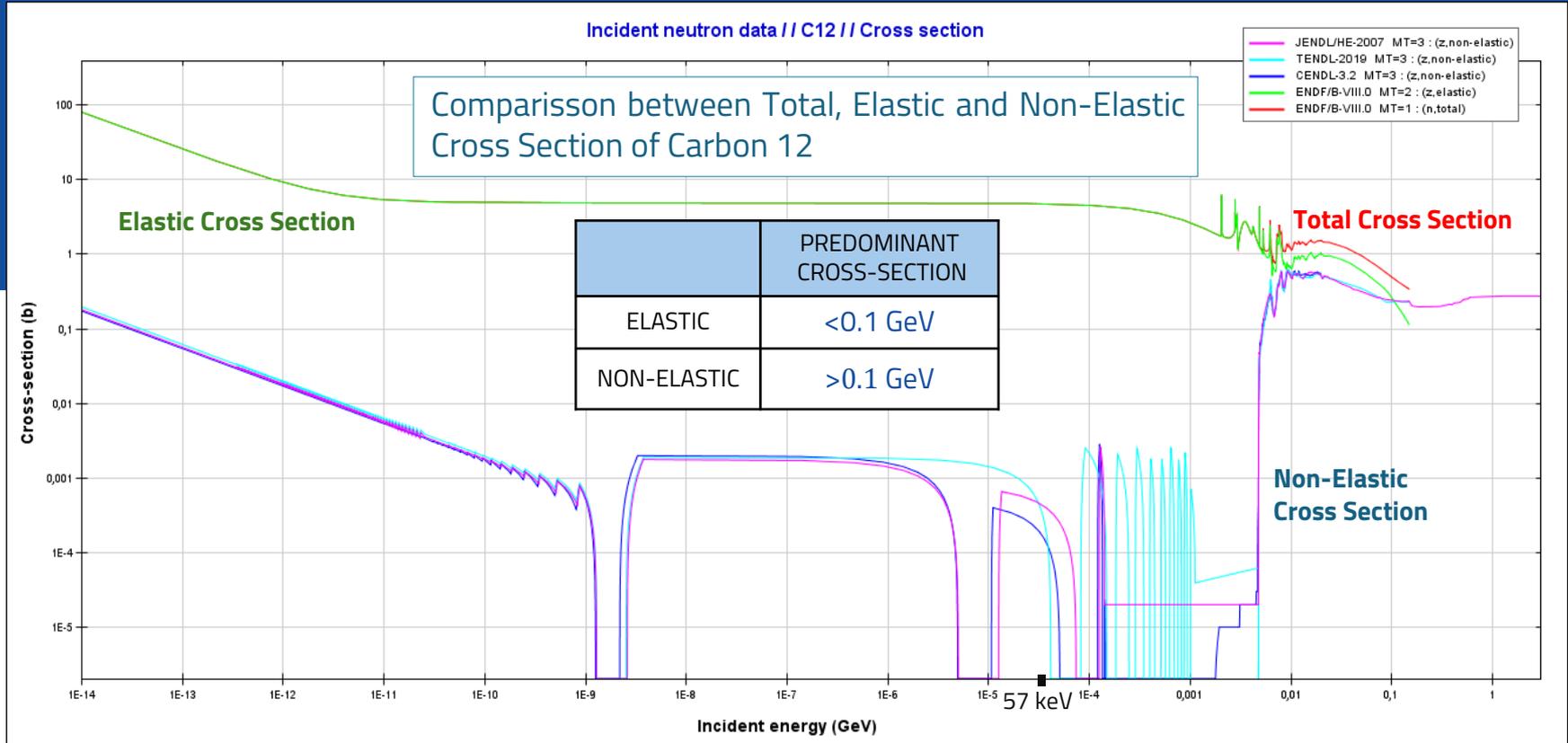


# Total 12C Cross Section

Incident neutron data // C12 //



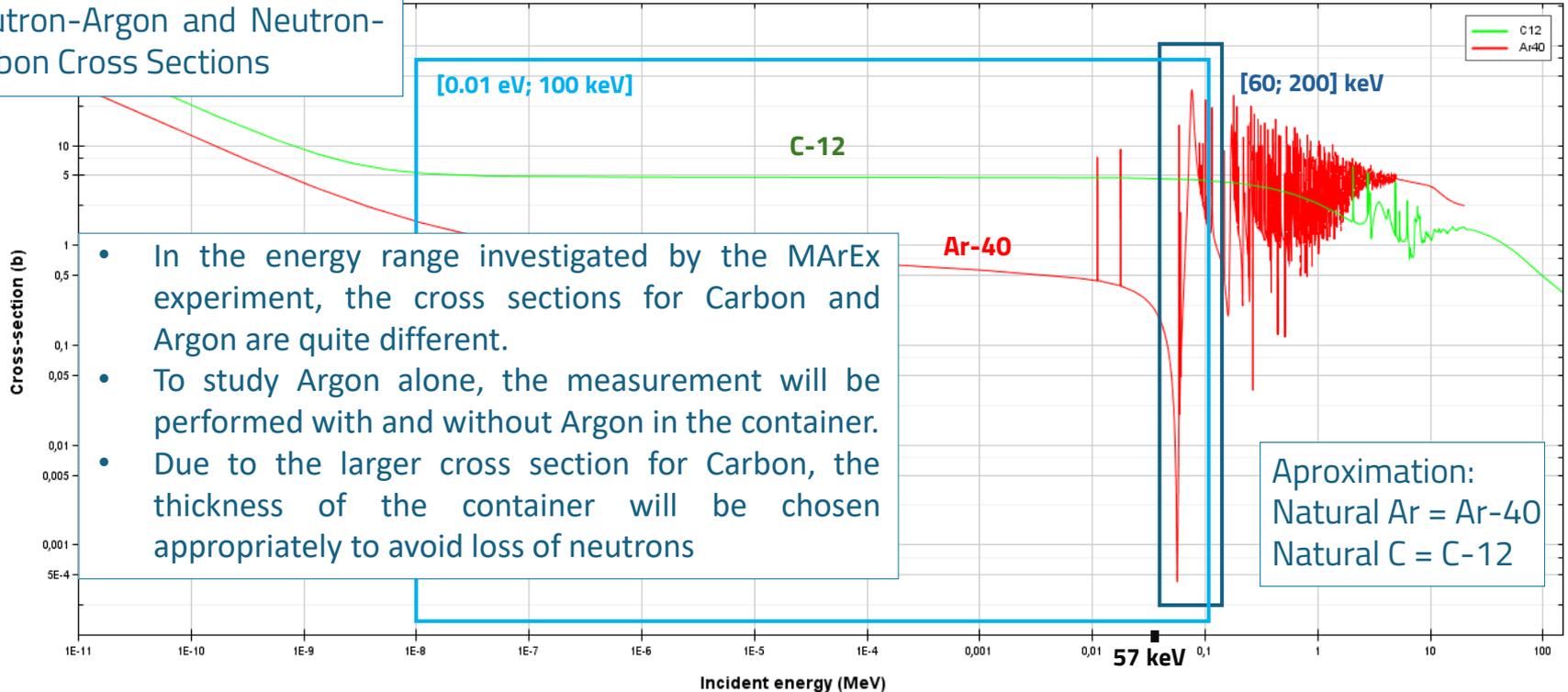
# Total, Elastic and Non-Elastic 12C Cross Section



# Total $^{40}\text{Ar}$ Cross Section vs $^{12}\text{C}$ Cross Section

Comparison between Neutron-Argon and Neutron-Carbon Cross Sections

Incident neutron data / ENDF/B-VIII.0 // MT=1 : (n,total) / Cross section



- In the energy range investigated by the MArEx experiment, the cross sections for Carbon and Argon are quite different.
- To study Argon alone, the measurement will be performed with and without Argon in the container.
- Due to the larger cross section for Carbon, the thickness of the container will be chosen appropriately to avoid loss of neutrons

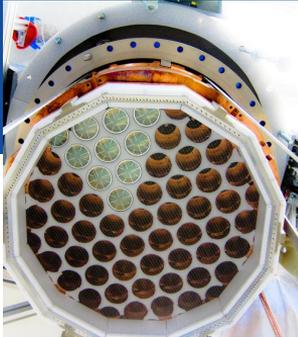
Aproximation:  
Natural Ar = Ar-40  
Natural C = C-12

## CONCLUSIONS

- I have shown that for natural abundance targets the **dominant interactions** are on the most abundant isotopes:  **$^{40}\text{Ar}$**  and  **$^{12}\text{C}$** .
- For neutron's interactions on **Carbon** targets, theoretical models agree very well with experimental data over the energy range, with the exception of energies below 10 meV, which however are less relevant for the MArEx experiment.
- Due to this well known behaviour **Carbon is a good material for building containers** + neutron interactions on Carbon can be used as a reference.

## CONCLUSIONS

- For neutron's interactions on **Argon** targets, the theoretical models well represent the experimental data above 10 keV, with the exception of TENDL-2019. However, TENDL-2019 shows a better agreement with experimental data for energies below 100 meV.
- It is important to investigate the energy range below 10 keV (lack of experimental data) and the resonance region (disagreement between old and new experimental data).



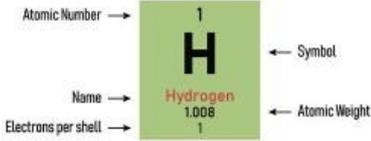
# Thank you for the attention!

## Any questions?

You can find me at [fc58292@alunos.fc.ul.pt](mailto:fc58292@alunos.fc.ul.pt)

# 5. Backup

1 IA <b>H</b> Hydrogen 1.008 1	2 IIA <b>He</b> Helium 4.0026 2											13 IIIA <b>B</b> Boron 10.81 2-3	14 IIIA <b>C</b> Carbon 12.011 2-4	15 VA <b>N</b> Nitrogen 14.007 2-5	16 VIA <b>O</b> Oxygen 15.999 2-4	17 VIIA <b>F</b> Fluorine 18.998 2-7	18 VIIIA <b>Ne</b> Neon 20.180 2-8
3 <b>Li</b> Lithium 6.94 2-1	4 <b>Be</b> Beryllium 9.012 2-2											13 <b>Al</b> Aluminium 26.982 2-3-3	14 <b>Si</b> Silicon 28.085 2-4-4	15 <b>P</b> Phosphorus 30.974 2-3-5	16 <b>S</b> Sulfur 32.06 2-3-4	17 <b>Cl</b> Chlorine 35.45 2-3-7	18 <b>Ar</b> Argon 39.948 2-8-8
11 <b>Na</b> Sodium 22.98976928 2-8-1	12 <b>Mg</b> Magnesium 24.305 2-8-2	3 IIIB <b>Sc</b> Scandium 44.955908 2-4-5-2	4 IVB <b>Ti</b> Titanium 47.867 2-4-8-2	5 VB <b>V</b> Vanadium 50.9415 2-4-5-2	6 VIB <b>Cr</b> Chromium 51.9961 2-4-5-1	7 VIIB <b>Mn</b> Manganese 54.938044 2-4-5-2	8 VIII <b>Fe</b> Iron 55.845 2-4-5-2	9 VIII <b>Co</b> Cobalt 58.933 2-4-5-2	10 VIII <b>Ni</b> Nickel 58.693 2-4-5-2	11 IB <b>Cu</b> Copper 63.546 2-4-5-2	12 IIB <b>Zn</b> Zinc 65.38 2-4-5-2	13 <b>Ga</b> Gallium 69.723 2-8-3	14 <b>Ge</b> Germanium 72.630 2-8-4	15 <b>As</b> Arsenic 74.922 2-8-5	16 <b>Se</b> Selenium 78.971 2-8-6	17 <b>Br</b> Bromine 79.904 2-8-7	18 <b>Kr</b> Krypton 83.798 2-4-8-6
19 <b>K</b> Potassium 39.0983 2-8-8-1	20 <b>Ca</b> Calcium 40.078 2-8-8-2	21 <b>Sc</b> Scandium 44.955908 2-6-8-2	22 <b>Ti</b> Titanium 47.867 2-6-8-2	23 <b>V</b> Vanadium 50.9415 2-6-8-2	24 <b>Cr</b> Chromium 51.9961 2-6-8-1	25 <b>Mn</b> Manganese 54.938044 2-6-8-2	26 <b>Fe</b> Iron 55.845 2-6-8-2	27 <b>Co</b> Cobalt 58.933 2-6-8-2	28 <b>Ni</b> Nickel 58.693 2-6-8-2	29 <b>Cu</b> Copper 63.546 2-6-8-2	30 <b>Zn</b> Zinc 65.38 2-6-8-2	31 <b>Ga</b> Gallium 69.723 2-8-3	32 <b>Ge</b> Germanium 72.630 2-8-4	33 <b>As</b> Arsenic 74.922 2-8-5	34 <b>Se</b> Selenium 78.971 2-8-6	35 <b>Br</b> Bromine 79.904 2-8-7	36 <b>Kr</b> Krypton 83.798 2-6-8-6
37 <b>Rb</b> Rubidium 85.4678 2-8-18-6-1	38 <b>Sr</b> Strontium 87.62 2-8-18-6-2	39 <b>Y</b> Yttrium 88.90584 2-6-18-2	40 <b>Zr</b> Zirconium 91.224 2-6-18-2	41 <b>Nb</b> Niobium 92.90637 2-4-8-10-1	42 <b>Mo</b> Molybdenum 95.95 2-6-8-10-2	43 <b>Tc</b> Technetium 98 2-4-8-10-2	44 <b>Ru</b> Ruthenium 101.07 2-6-8-10-1	45 <b>Rh</b> Rhodium 102.91 2-6-8-10-1	46 <b>Pd</b> Palladium 106.42 2-4-8-10-2	47 <b>Ag</b> Silver 107.87 2-4-8-10-1	48 <b>Cd</b> Cadmium 112.41 2-4-8-10-2	49 <b>In</b> Indium 114.82 2-4-8-10-3	50 <b>Sn</b> Tin 118.71 2-4-8-10-4	51 <b>Sb</b> Antimony 121.76 2-4-8-10-3	52 <b>Te</b> Tellurium 127.60 2-4-8-10-4	53 <b>I</b> Iodine 126.90 2-4-8-10-7	54 <b>Xe</b> Xenon 131.29 2-6-8-10-6
55 <b>Cs</b> Cesium 132.90545196 2-8-18-32-1-1	56 <b>Ba</b> Barium 137.327 2-4-8-18-6-2	57-71 Lanthanides	72 <b>Hf</b> Hafnium 178.49 2-6-18-32-10-2	73 <b>Ta</b> Tantalum 180.94788 2-4-8-18-32-10-1	74 <b>W</b> Tungsten 183.84 2-4-8-32-10-3	75 <b>Re</b> Rhenium 186.21 2-4-8-32-10-3	76 <b>Os</b> Osmium 190.23 2-4-8-32-10-4	77 <b>Ir</b> Iridium 192.22 2-4-8-32-10-4	78 <b>Pt</b> Platinum 195.084 2-4-8-32-10-1	79 <b>Au</b> Gold 196.97 2-4-8-32-10-1	80 <b>Hg</b> Mercury 200.59 2-4-8-32-10-2	81 <b>Tl</b> Thallium 204.38 2-4-8-32-10-3	82 <b>Pb</b> Lead 207.2 2-4-8-32-10-4	83 <b>Bi</b> Bismuth 208.98 2-4-8-32-10-1	84 <b>Po</b> Polonium 209 2-4-8-32-10-4	85 <b>At</b> Astatine 210 2-4-8-32-10-7	86 <b>Rn</b> Radon 222 2-6-8-32-10-6
87 <b>Fr</b> Francium 223 2-4-8-32-10-1-1	88 <b>Ra</b> Radium 226 2-4-8-32-10-2	89-103 Actinides	104 <b>Rf</b> Rutherfordium 261 2-6-18-32-10-2	105 <b>Db</b> Dubnium 262 2-4-8-32-10-1	106 <b>Sg</b> Seaborgium 269 2-4-8-32-10-3	107 <b>Bh</b> Bohrium 270 2-4-8-32-10-3	108 <b>Hs</b> Hassium 277 2-4-8-32-10-4	109 <b>Mt</b> Meitnerium 276 2-4-8-32-10-5	110 <b>Ds</b> Darmstadtium 285 2-4-8-32-10-1	111 <b>Rg</b> Roentgenium 282 2-4-8-32-10-2	112 <b>Cn</b> Copernicium 285 2-4-8-32-10-2	113 <b>Nh</b> Nihonium 284 2-4-8-32-10-3	114 <b>Fl</b> Flerovium 289 2-4-8-32-10-4	115 <b>Mc</b> Moscovium 290 2-4-8-32-10-5	116 <b>Lv</b> Livermorium 293 2-4-8-32-10-6	117 <b>Ts</b> Tennessine 294 2-4-8-32-10-7	118 <b>Og</b> Oganesson 294 2-6-8-32-10-8



State of matter (color of name)  
GAS LIQUID SOLID UNKNOWN

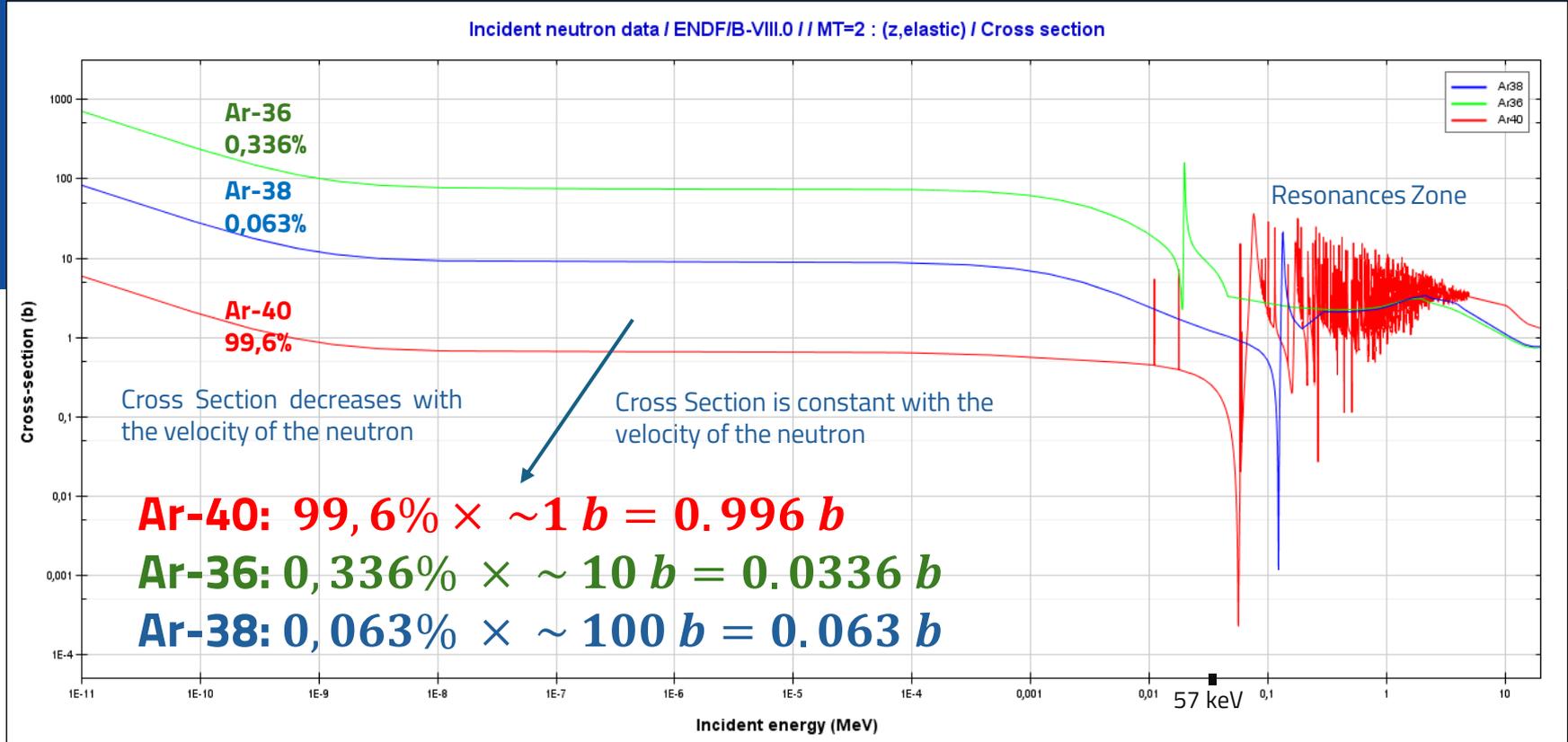
Subcategory in the metal-metalloid-nonmetal trend (color of background)

- Alkali metals
- Alkaline earth metals
- Transition metals
- Lanthanides
- Actinides
- Post-transition metals
- Metalloids
- Reactive nonmetals
- Noble gases

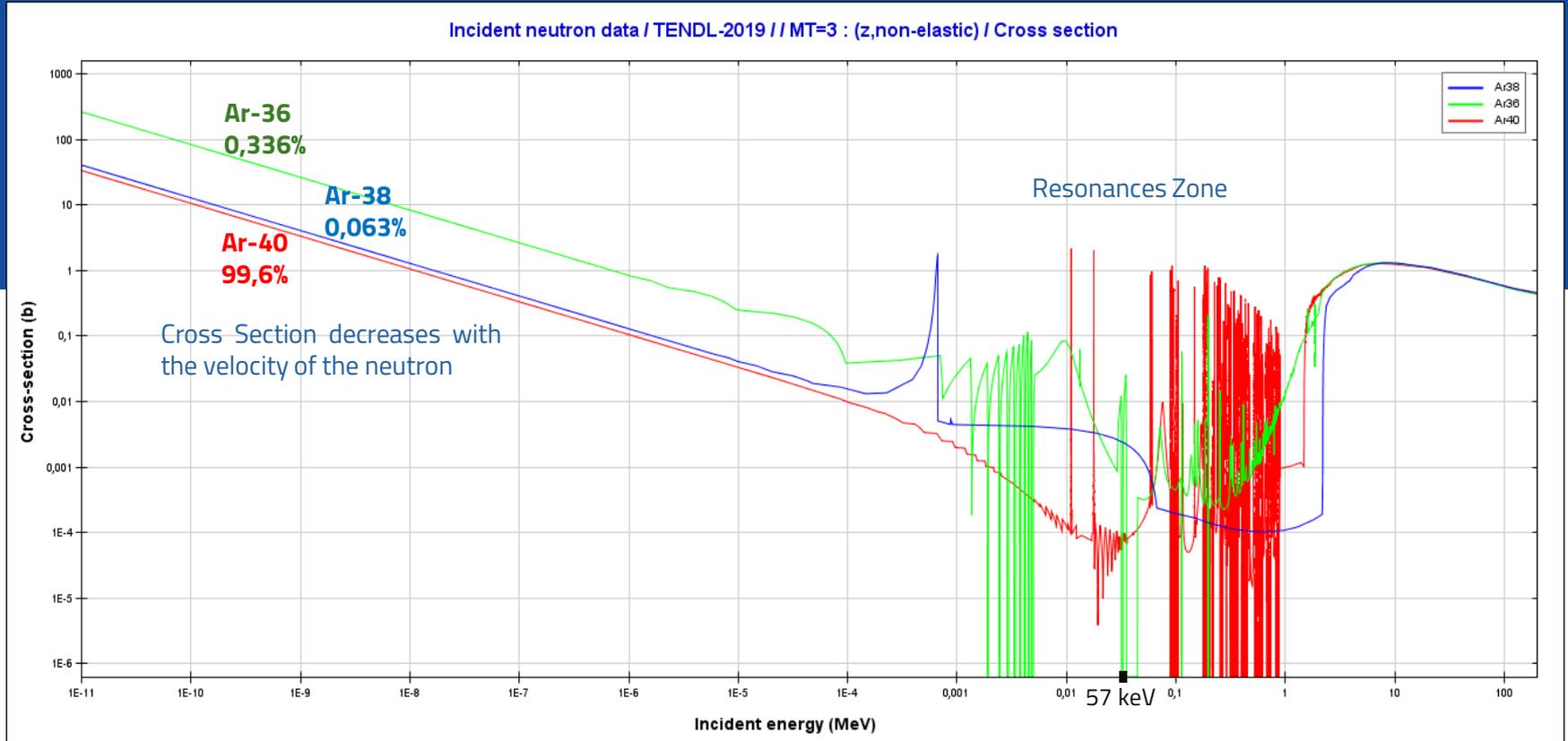
Unknown chemical properties

57 <b>La</b> Lanthanum 138.91 2-4-8-10-4-2	58 <b>Ce</b> Cerium 140.12 2-4-8-10-4-1	59 <b>Pr</b> Praseodymium 140.91 2-6-8-10-4-2	60 <b>Nd</b> Neodymium 144.24 2-4-8-10-4-2	61 <b>Pm</b> Promethium 145 2-4-8-10-4-2	62 <b>Sm</b> Samarium 150.36 2-4-8-10-4-2	63 <b>Eu</b> Europium 151.96 2-6-8-10-4-2	64 <b>Gd</b> Gadolinium 157.25 2-4-8-10-4-2	65 <b>Tb</b> Terbium 158.93 2-6-8-10-4-2	66 <b>Dy</b> Dysprosium 162.5 2-4-8-10-4-2	67 <b>Ho</b> Holmium 164.93 2-4-8-10-4-2	68 <b>Er</b> Erbium 167.26 2-4-8-10-4-2	69 <b>Tm</b> Thulium 168.93 2-6-8-10-4-2	70 <b>Yb</b> Ytterbium 173.05 2-6-8-10-4-2	71 <b>Lu</b> Lutetium 174.97 2-6-8-10-4-2
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# Elastic $^{40}\text{Ar}$ , $^{36}\text{Ar}$ and $^{38}\text{Ar}$ Cross Section (backup)

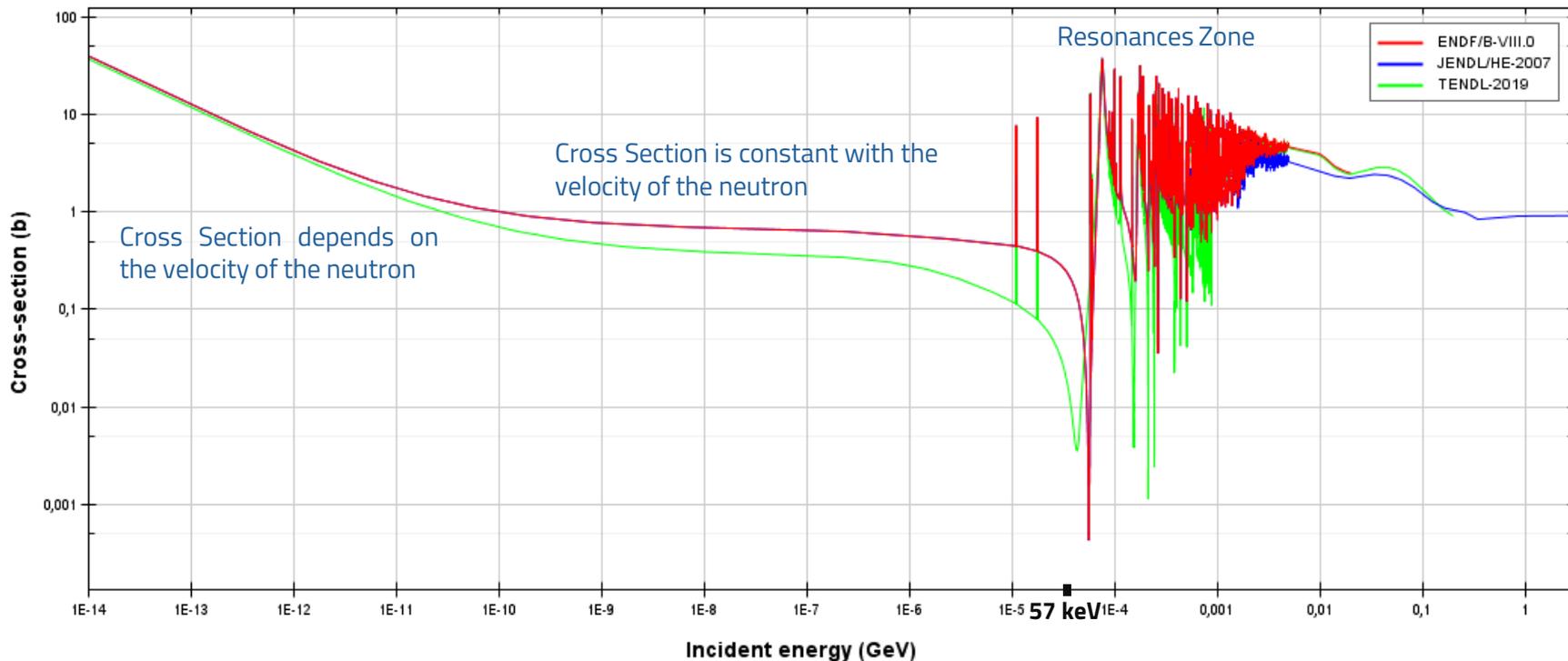


# Non-Elastic $^{40}\text{Ar}$ , $^{36}\text{Ar}$ and $^{38}\text{Ar}$ Cross Section (backup)



# Total $^{40}\text{Ar}$ Cross Section

Incident neutron data // Ar40 / MT=1 : (n,total) / Cross section

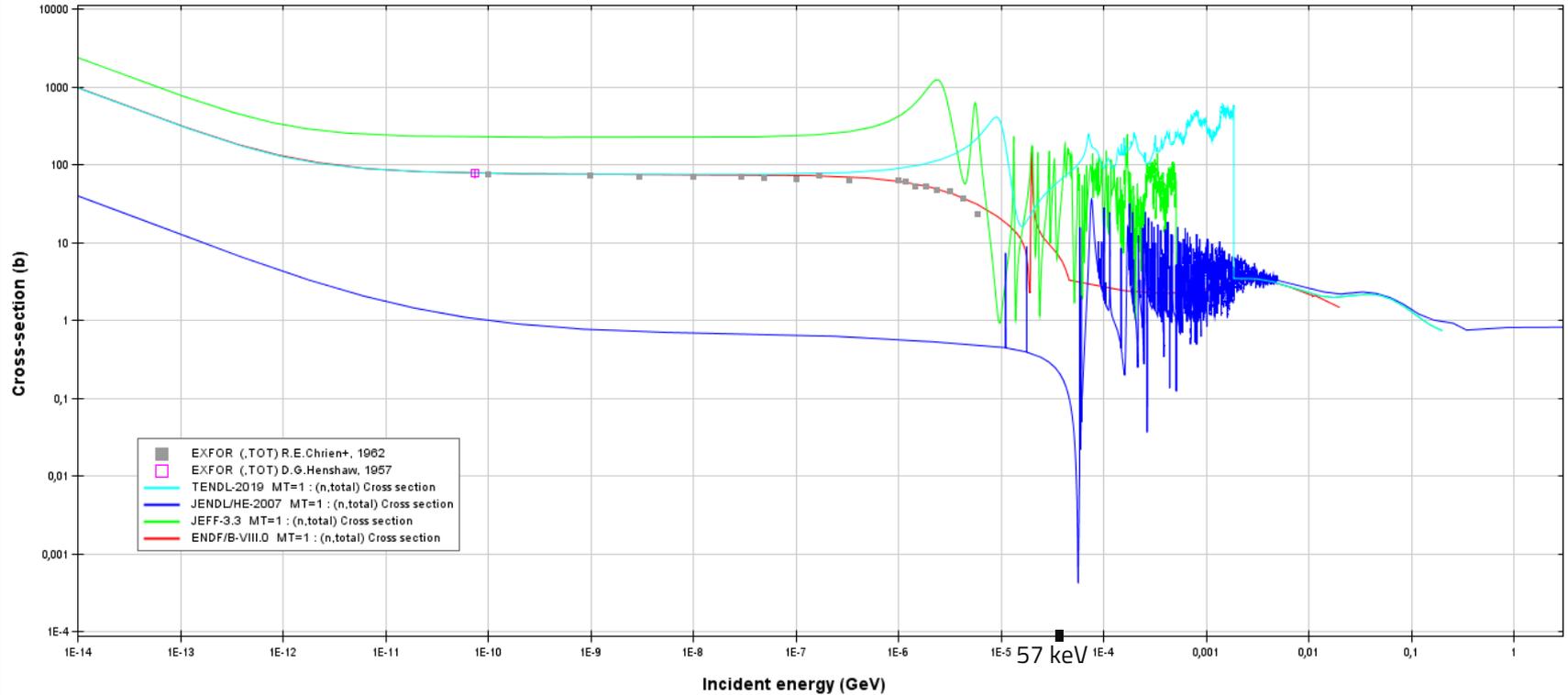


**3.1.2**

**Argon 36 (backup)**

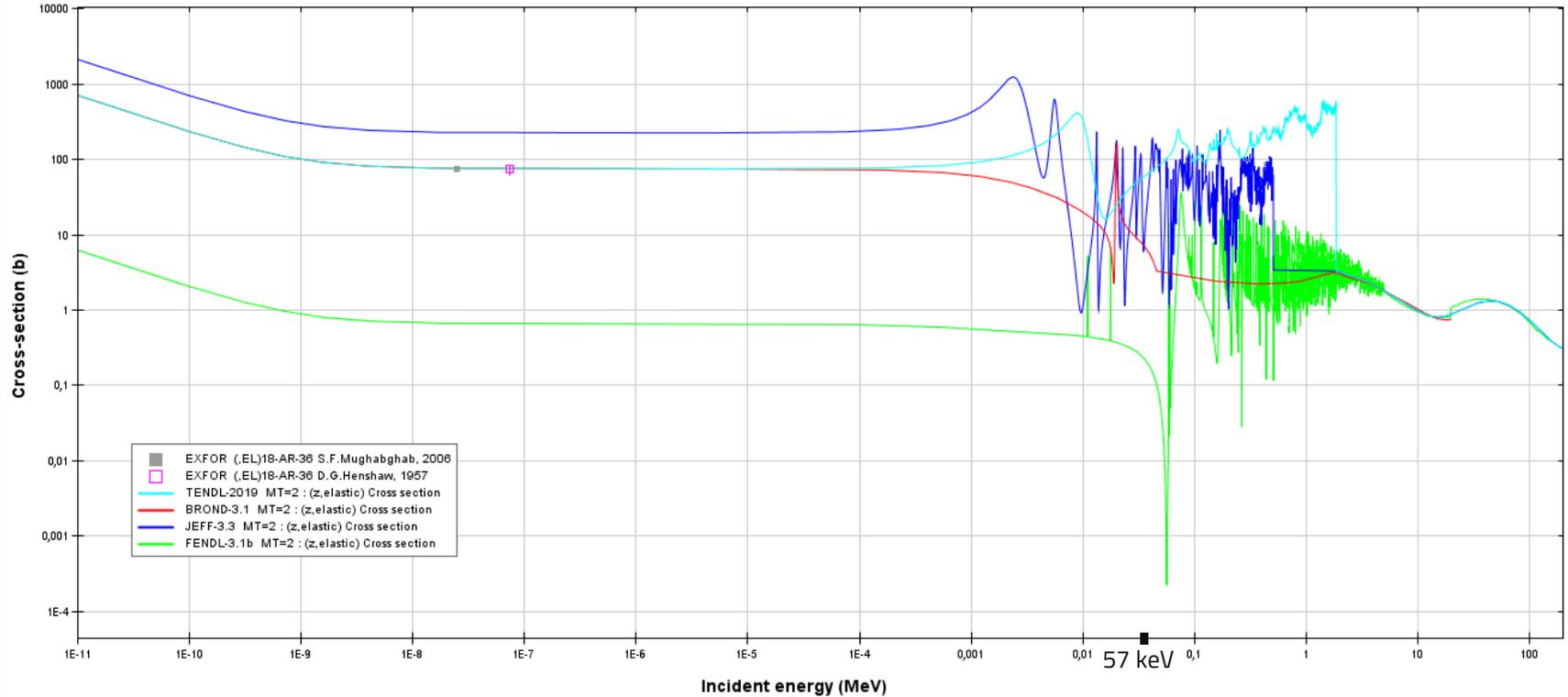
# Total $^{36}\text{Ar}$ Cross Section

Incident neutron data // Ar36 //



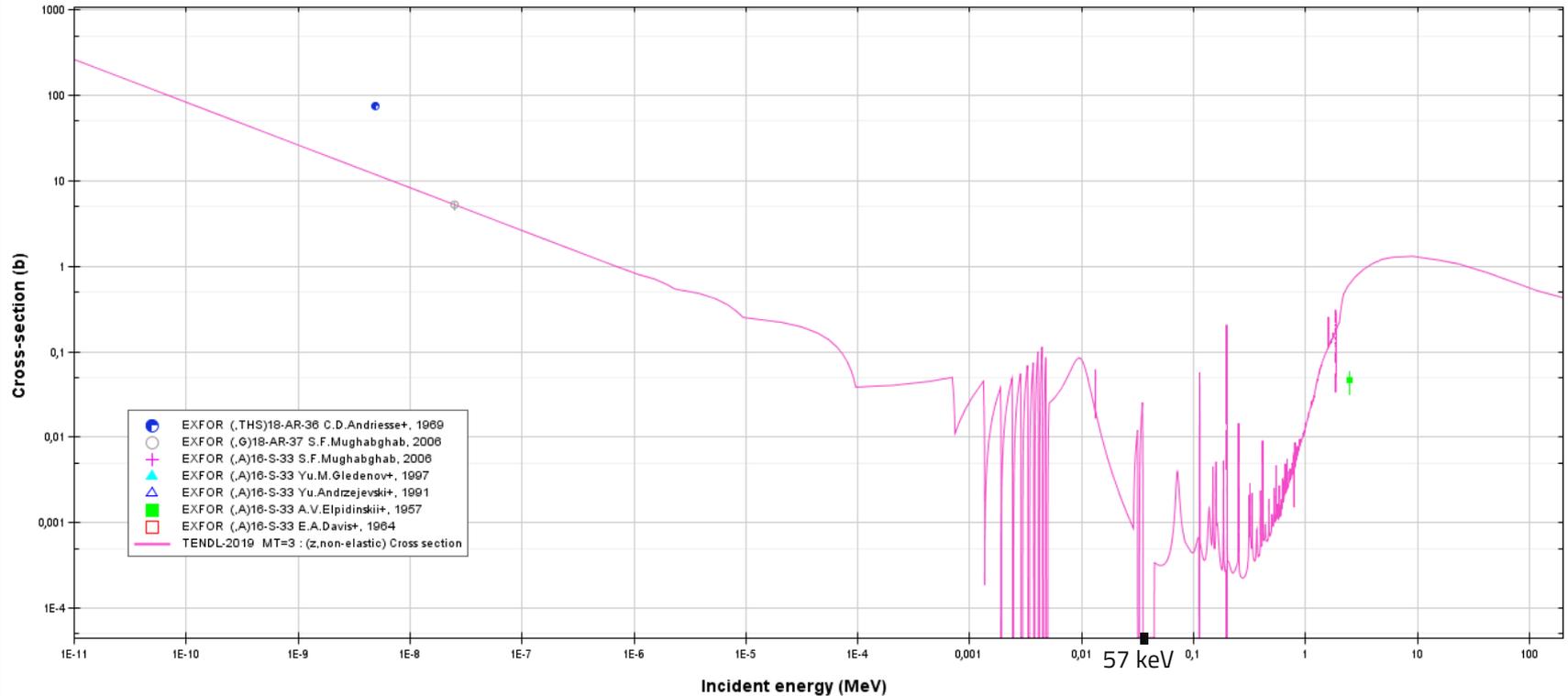
# Elastic $^{36}\text{Ar}$ Cross Section

Incident neutron data // Ar36 //



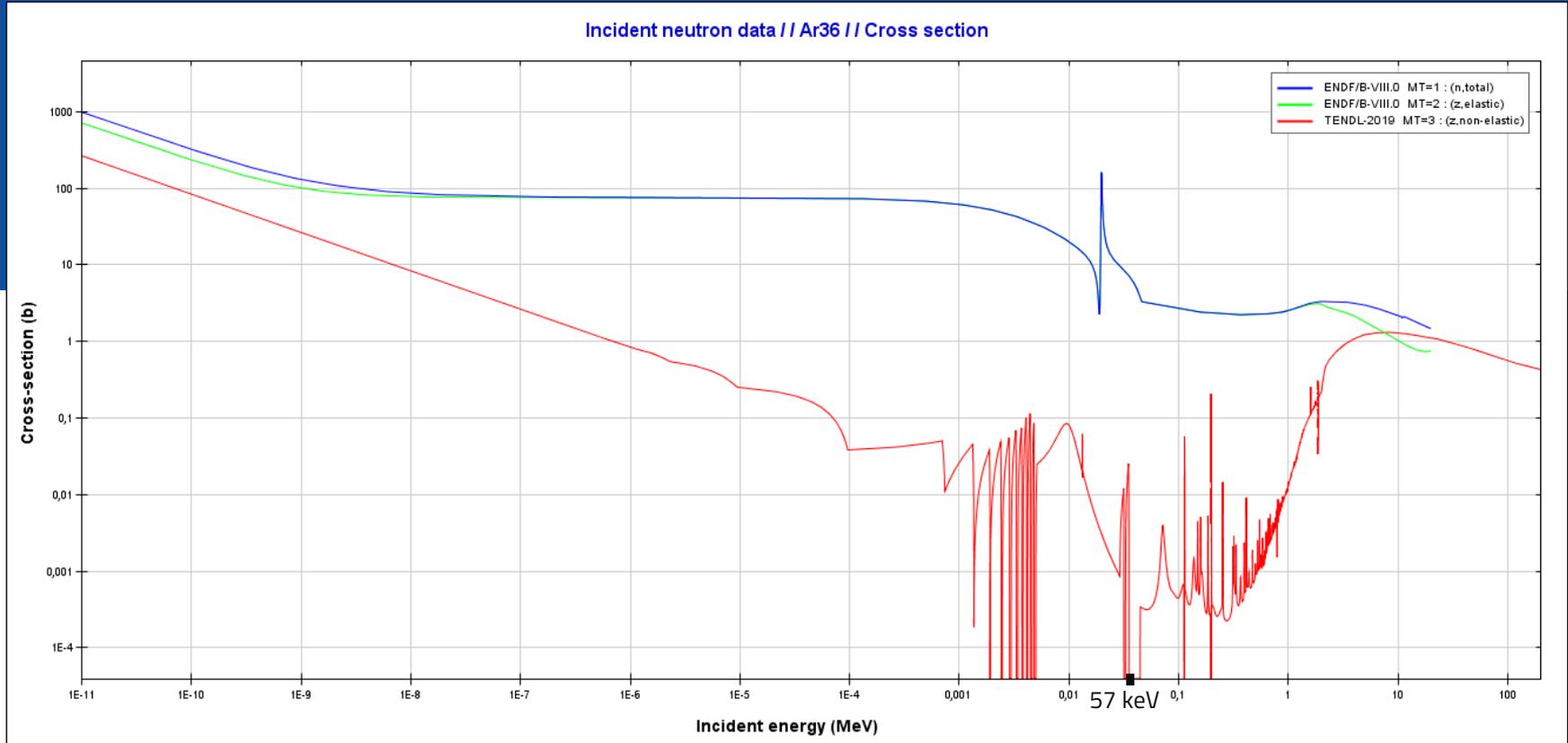
# Non-Elastic $^{36}\text{Ar}$ Cross Section

Incident neutron data // Ar36 //



ENERGY	EMISSION
[0.01; 0.1] eV	$n + \text{Ar-36} \rightarrow \alpha + \text{S-16}$
[0.01; 0.1] eV	$n + \text{Ar-36} \rightarrow \gamma + \text{Ar-37}$
[0.001; 0.01] eV	$n + \text{Ar-36} \rightarrow \text{Cl-40}$ THS...

# Total, Elastic and Non-Elastic $^{36}\text{Ar}$ Cross Section

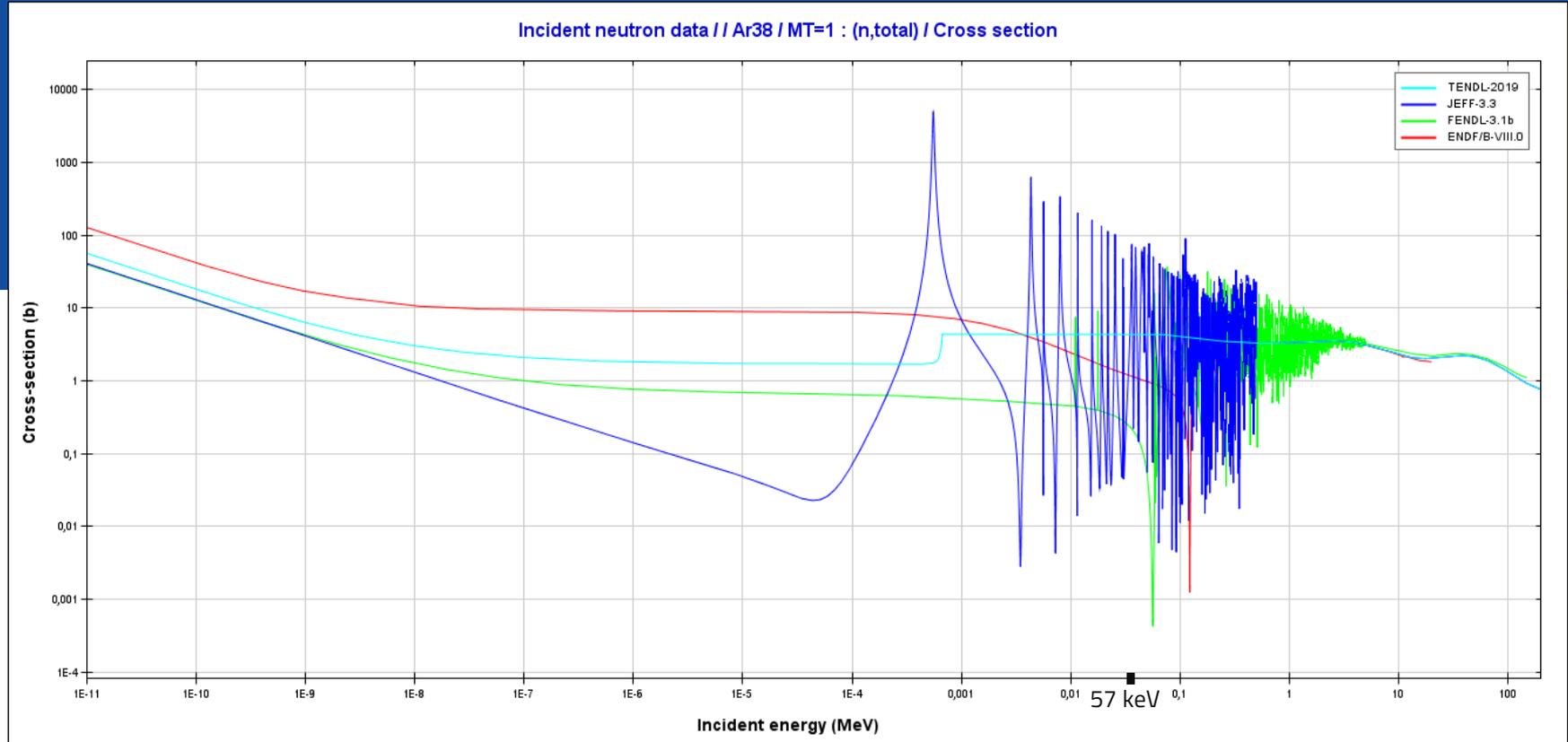


	PREDOMINANT CROSS-SECTION
ELASTIC	$[1 \times 10^{-11}; \pm 9]$ MeV
NON-ELASTIC	$[\pm 9; 100]$ MeV

**3.1.3**

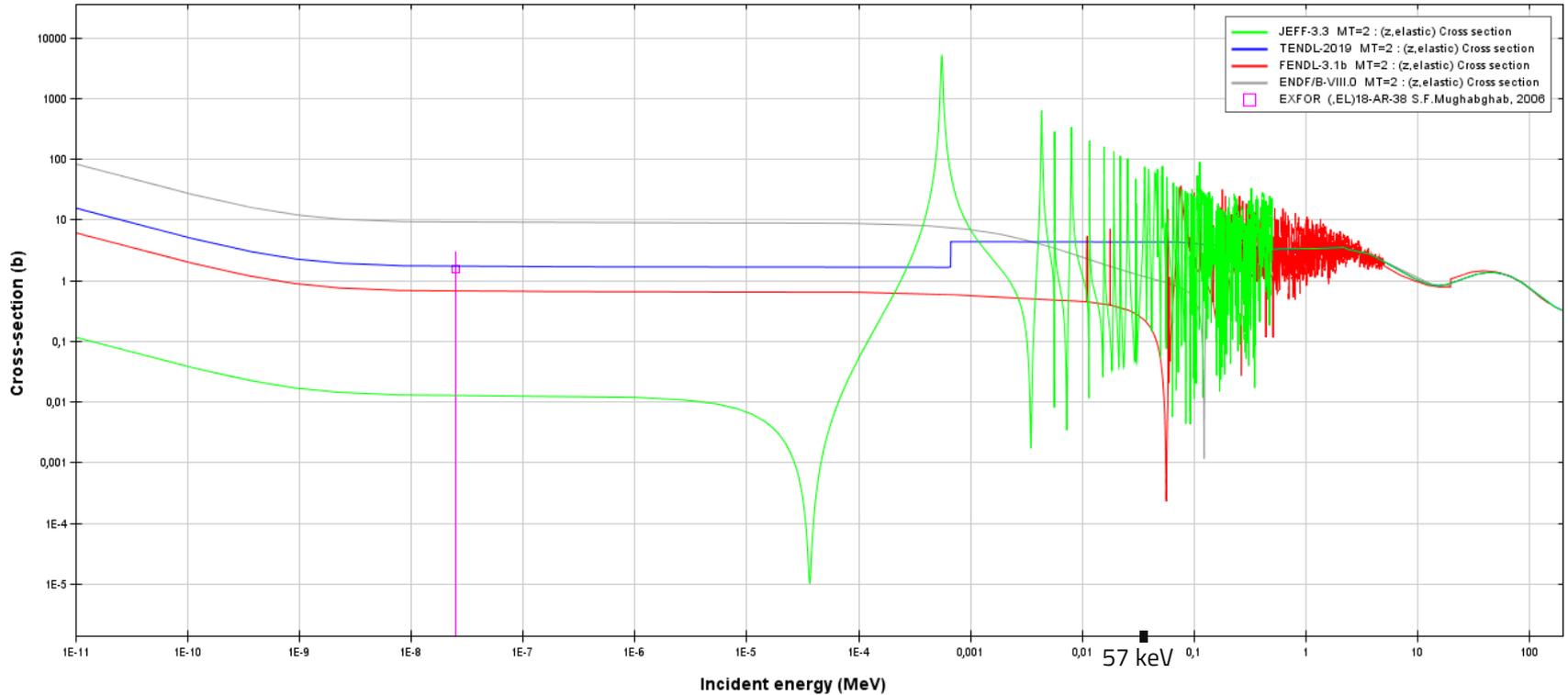
**Argon 38 (backup)**

# Total $^{38}\text{Ar}$ Cross Section



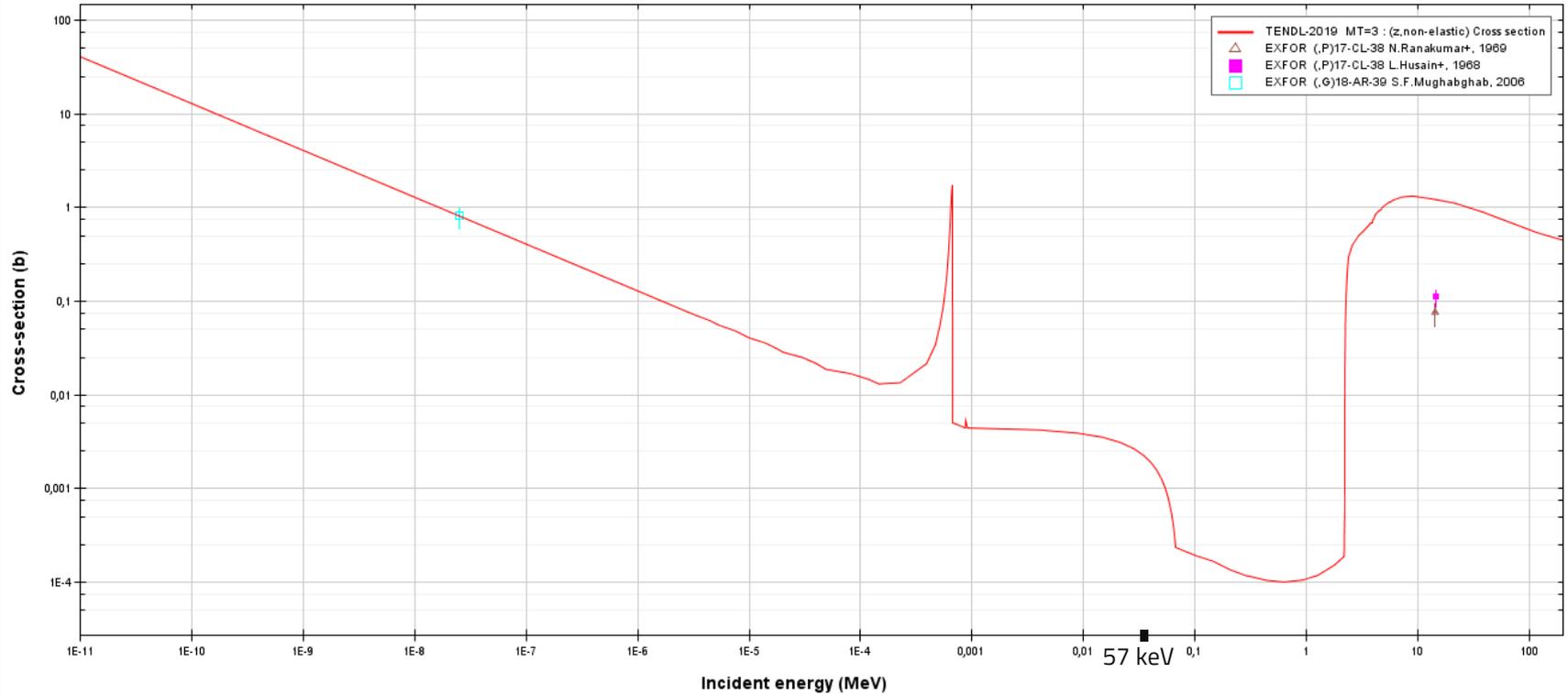
# Elastic 38 Cross Section

Incident neutron data // Ar38 //



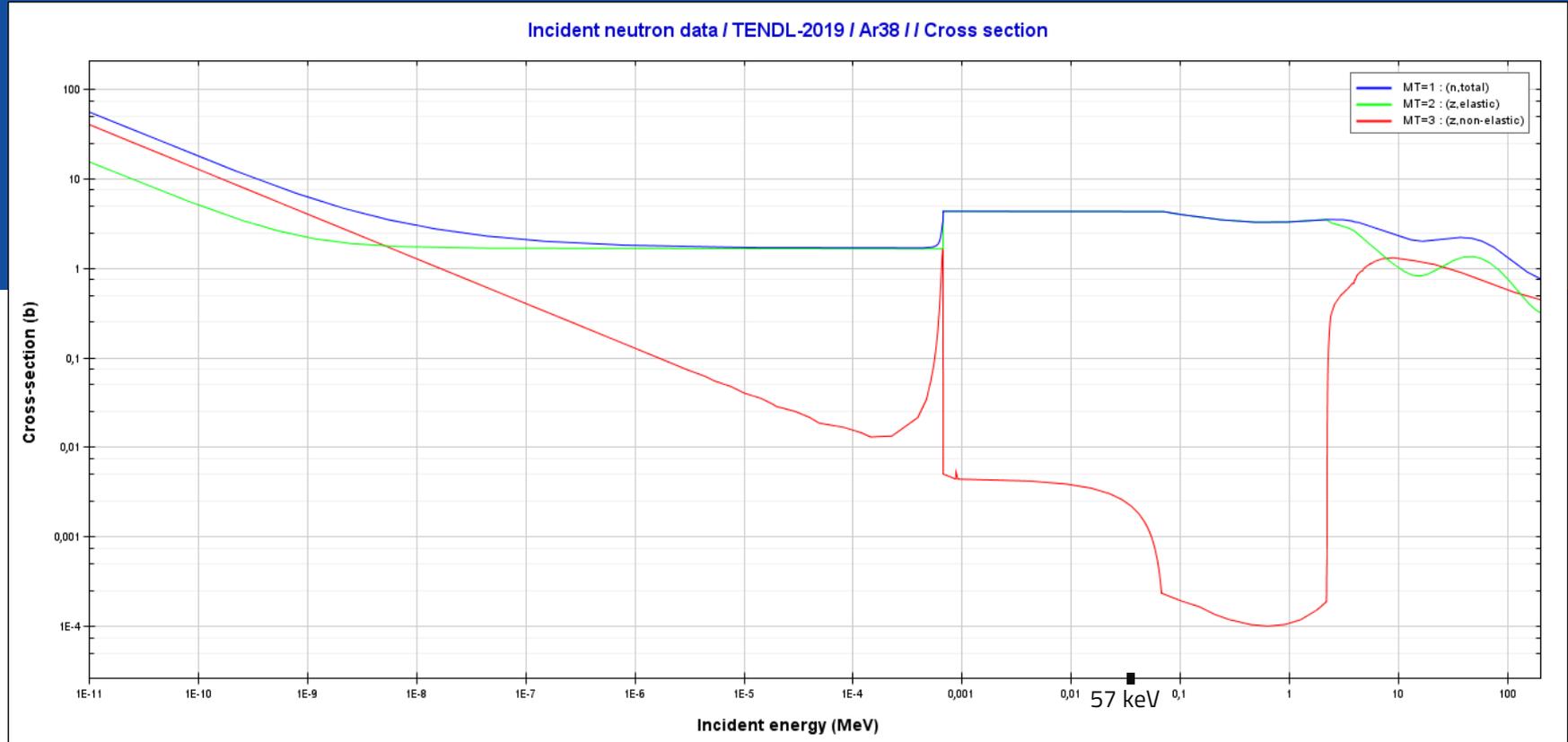
# Non-Elastic $^{38}\text{Ar}$ Cross Section

Incident neutron data // Ar38 //



ENERGY	EMISSION
[0.01; 0.1] eV	<b><math>n + \text{Ar-38} \rightarrow \gamma + \text{Ar-39}</math></b>
[10; 100] MeV	<b><math>n + \text{Ar-38} \rightarrow p + \text{Cl-38}</math></b>

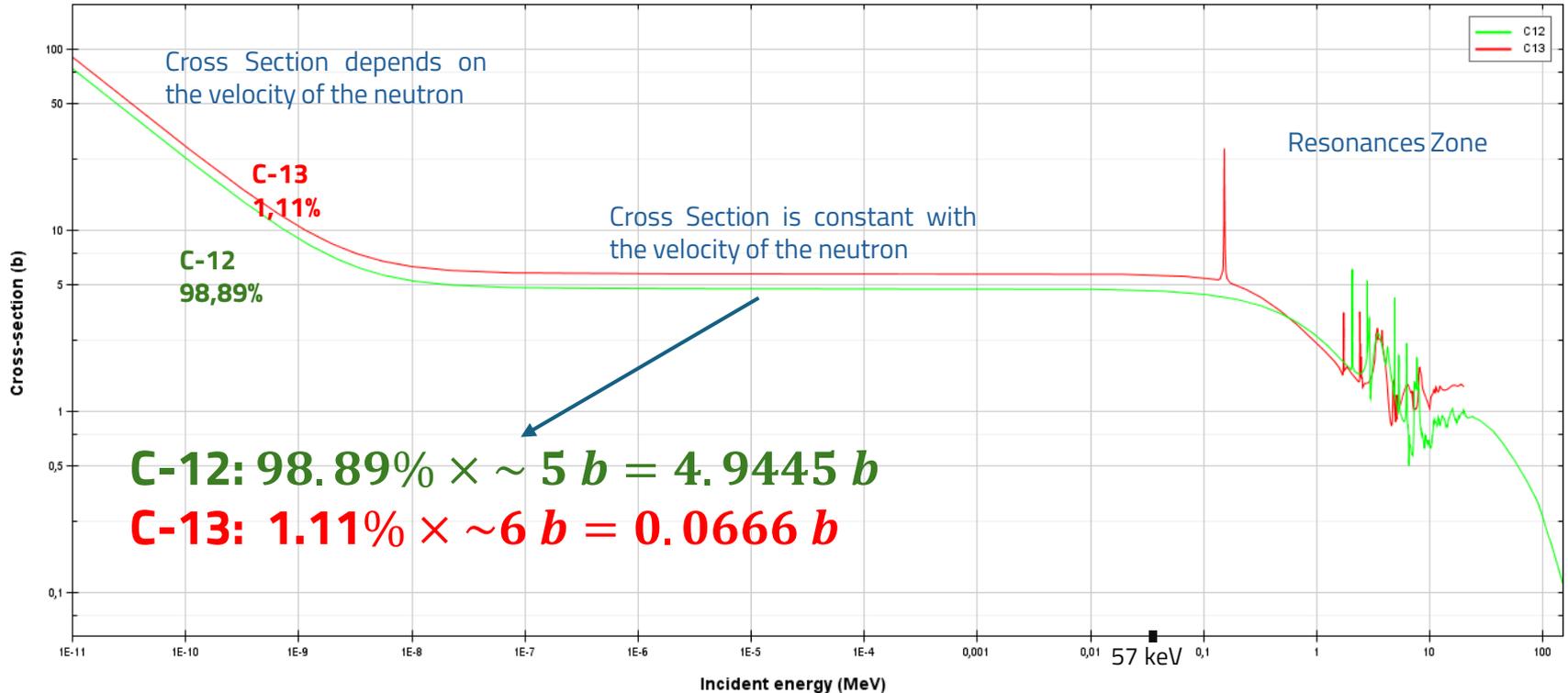
# Total, Elastic and Non-Elastic $^{38}\text{Ar}$ Cross Section



	PREDOMINANT CROSS-SECTION	
ELASTIC	$[\pm 7 \times 10^{-9}; \pm 9] \text{ MeV}$	$[\pm 40; \pm 100] \text{ MeV}$
NON-ELASTIC	$[1 \times 10^{-11}; \pm 7 \times 10^{-9}] \text{ MeV}$	$[\pm 9; \pm 40] \text{ MeV}$

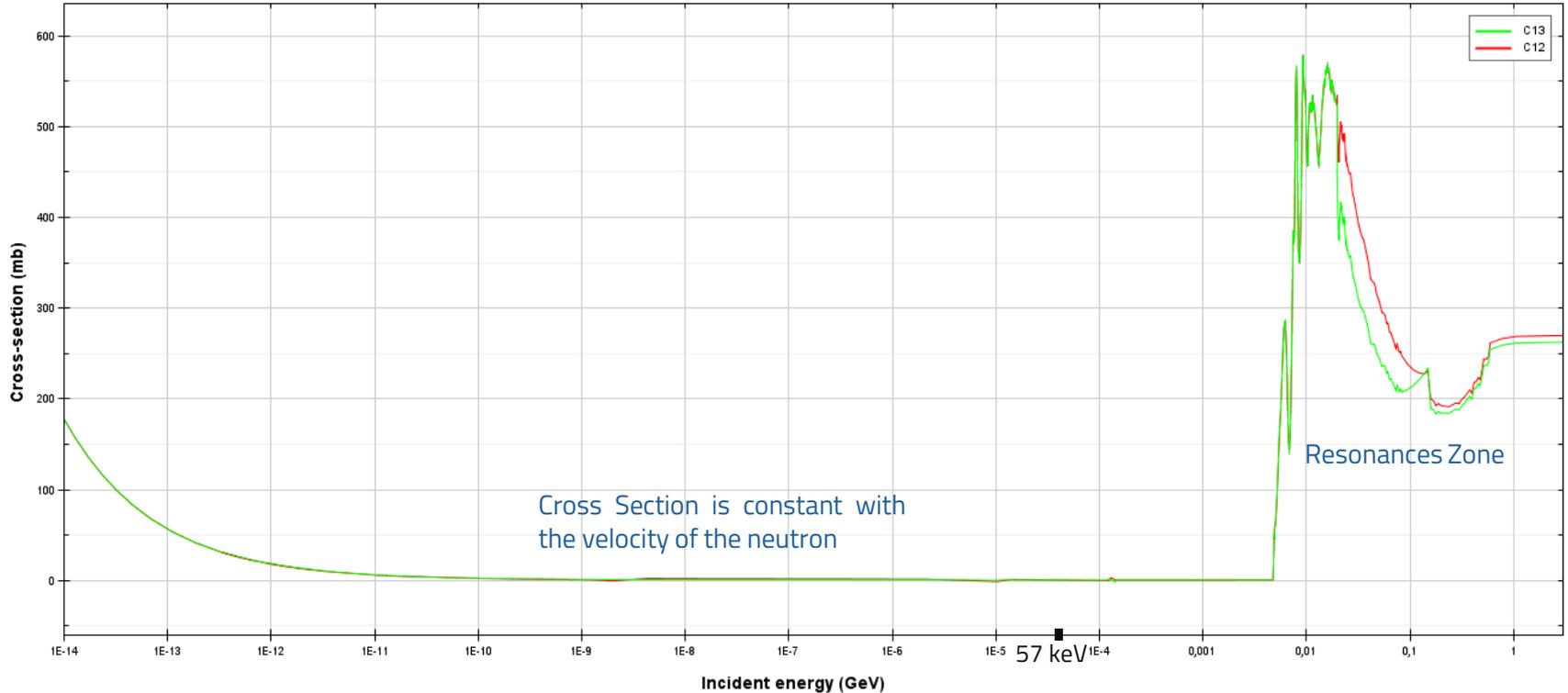
# Elastic 12C and 13C Cross Section (backup)

Incident neutron data / ENDF/B-VIII.0 // MT=2 : (z,elastic) / Cross section



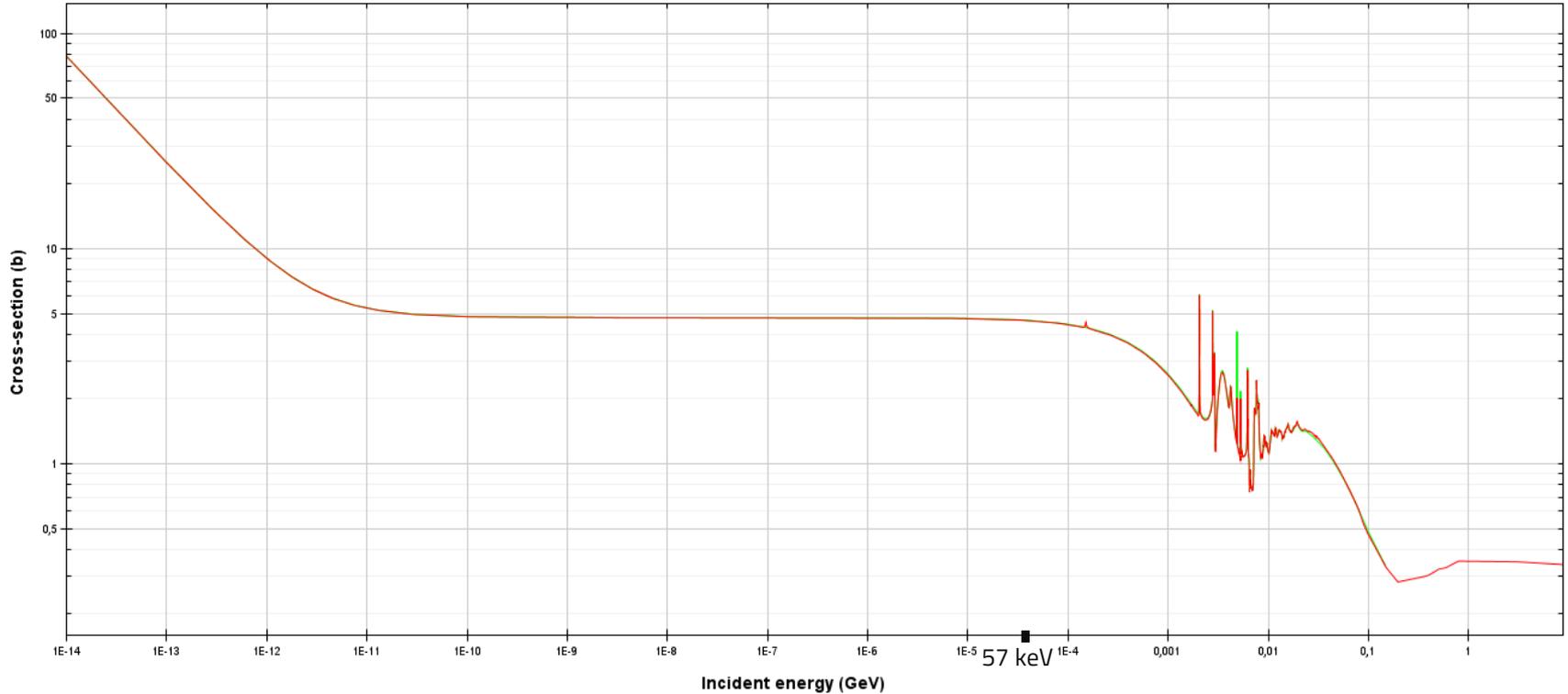
# Non-Elastic $^{12}\text{C}$ and $^{13}\text{C}$ Cross Section (backup)

Incident neutron data / JENDL/HE-2007 // MT=3 : (z,non-elastic) / Cross section



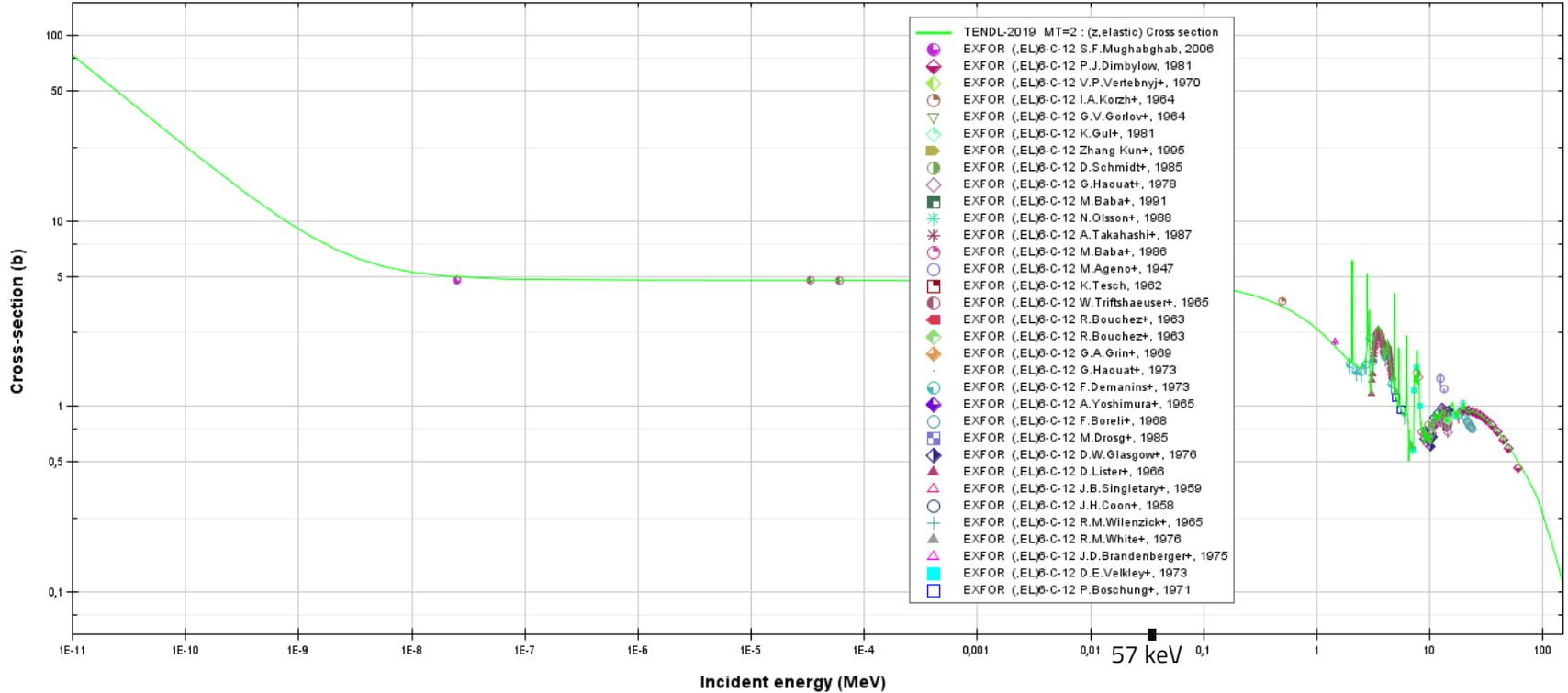
# Total $^{12}\text{C}$ Cross Section (backup)

Incident neutron data // C12 / MT=1 : (n,total) / Cross section



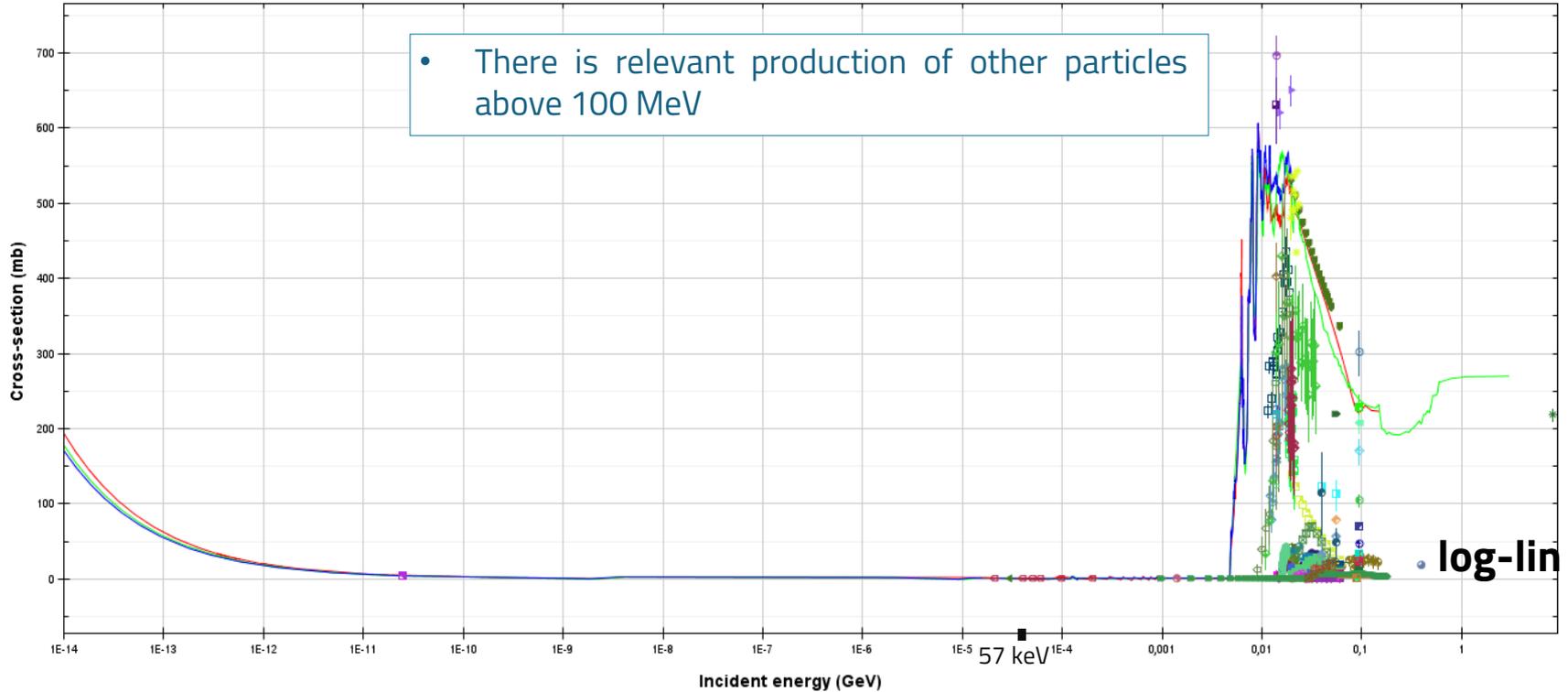
# Elastic 12C Cross Section (backup)

Incident neutron data // C12 //



# Non-Elastic $^{12}\text{C}$ Cross Section (backup)

Incident neutron data // C12 //



EMISSION	PRODUCCION
[0.01; 0.05] GeV	$n + \text{C-12} \rightarrow X + \text{H-0}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{H-1}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{H-2}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{H-3}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{He-3}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{He-4}$

[0.005; 0.1] GeV	$n + \text{C-12} \rightarrow n + 2\alpha + \text{He-4}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow \alpha + n + p + \text{H-3} + \text{He-4}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow 2\alpha + \text{He-5}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{He-6}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + p + n + \alpha + \text{Li-6}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow \alpha + \text{H-2} + \text{Li-7}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow \alpha + n + p + \text{Li-7}$

[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + p + \alpha + \text{Li-7}$
[0.001; 1] GeV	$n + \text{C-12} \rightarrow p + \alpha + \text{Li-8}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{Li-8}$
[0.05; 0.1] GeV	$n + \text{C-12} \rightarrow X + \text{Li-9}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow 2n + \alpha + \text{Be-7}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + \alpha + n + \text{Be-7}$
[0.001; 0.5] GeV	$n + \text{C-12} \rightarrow X + \text{Be-7}$

[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow \alpha + n + \text{Be-8}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + \alpha + \text{Be-8}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + p + \text{H-3} + \text{Be-8}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow \alpha + \text{Be-9}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow n + \text{H-3} + \text{Be-9}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow p + \text{H-3} + \text{Be-9}$
[0.01; 0.1] GeV	$n + \text{C-12} \rightarrow p + \text{H-2} + \text{Be-10}$

[0.05; 0.1] GeV	$n + C-12 \rightarrow X + B-8$
[0.01; 0.1] GeV	$n + C-12 \rightarrow 2n + H-2 + B-9$
[0.01; 0.1] GeV	$n + C-12 \rightarrow 2n + p + B-10$
[0.01; 0.1] GeV	$n + C-12 \rightarrow H-2 + n + B-10$
[0.01; 0.1] GeV	$n + C-12 \rightarrow n + H-2 + B-10$
[0.01; 0.1] GeV	$n + C-12 \rightarrow n + p + n + B-10$
[0.01; 0.1] GeV	$n + C-12 \rightarrow H-3 + B-10$

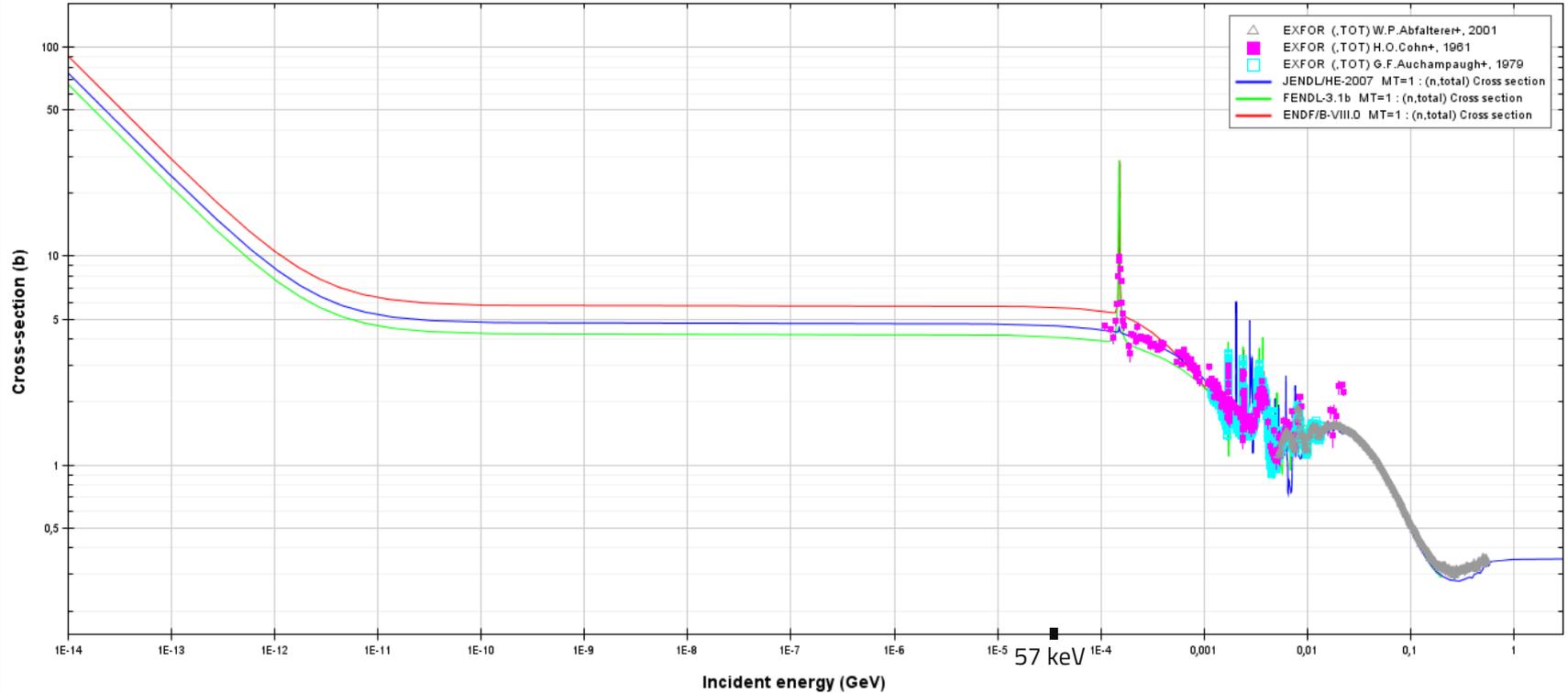
[0.01; 0.1] GeV	$n + C-12 \rightarrow H-2 + B-11$
[0.01; 0.1] GeV	$n + C-12 \rightarrow n + p + B-11$
[0.01; 0.1] GeV	$n + C-12 \rightarrow p + B-12$
[0.01; 0.1] GeV	$n + C-12 \rightarrow 3n + C-10$
[0.001; 1] GeV	$n + C-12 \rightarrow 2n + C-11$
[0.1; 10] meV	$n + C-12 \rightarrow THS + C-12$
[0.01; 0.1] eV/ [0.01; 1] MeV	$n + C-12 \rightarrow \gamma + C-13$

**3.2.2**

**Carbon 13 (backup)**

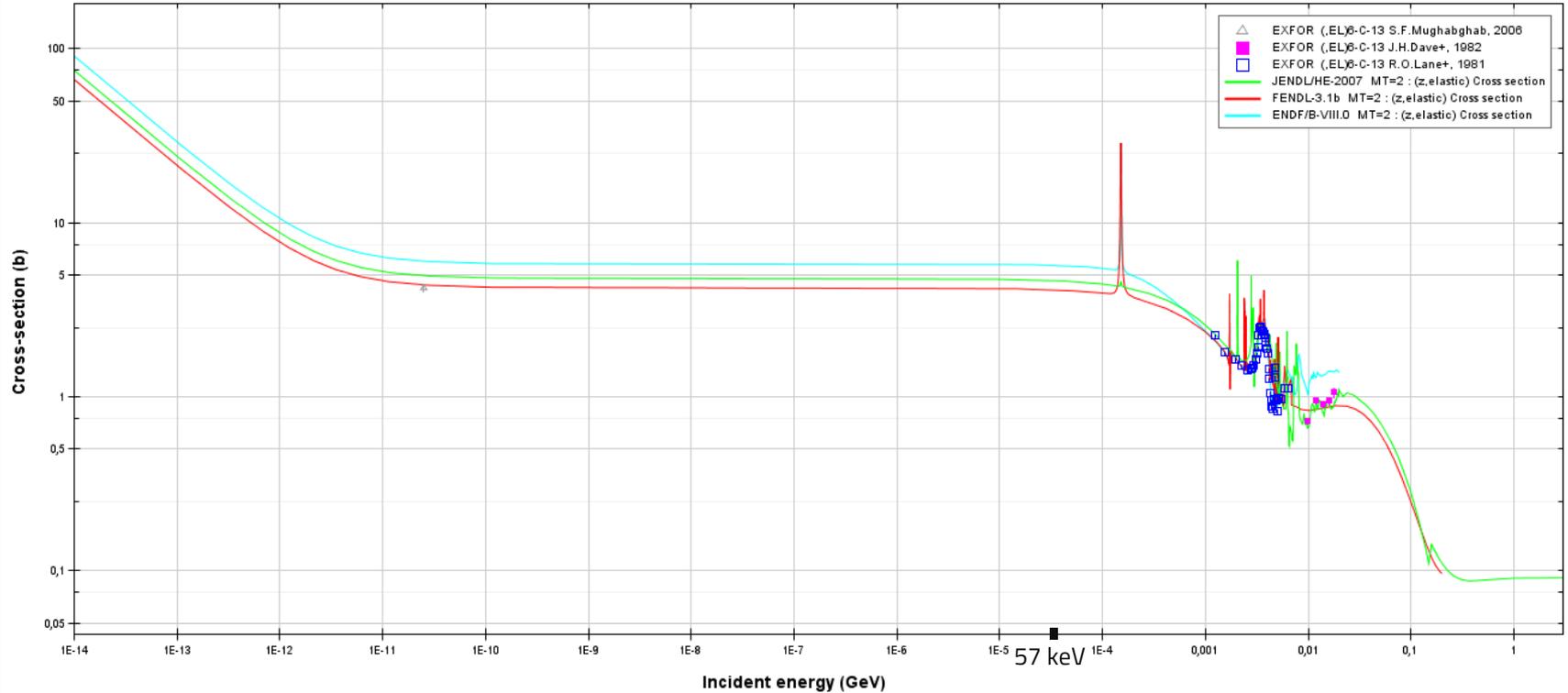
# Total $^{13}\text{C}$ Cross Section

Incident neutron data // C13 //



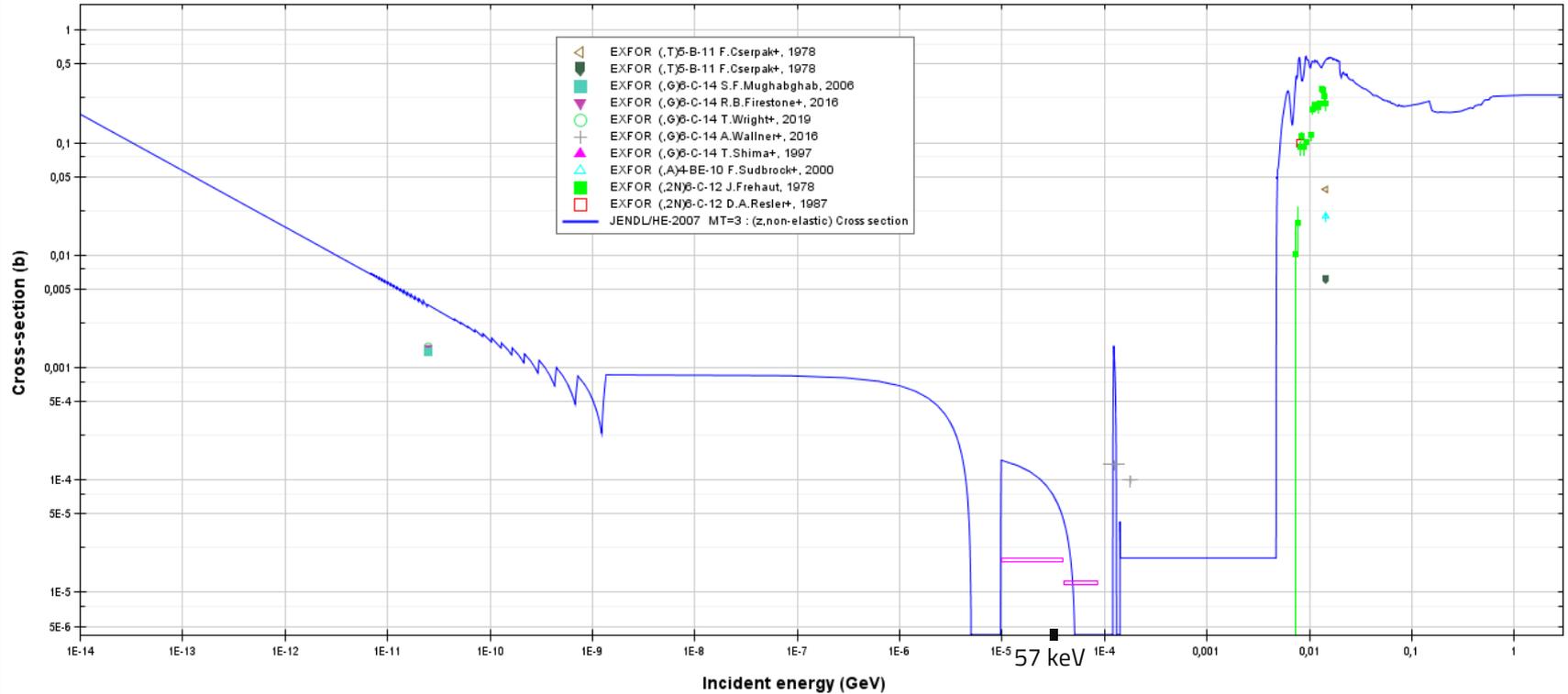
# Elastic $^{13}\text{C}$ Cross Section

Incident neutron data // C13 //



# Non-Elastic $^{13}\text{C}$ Cross Section

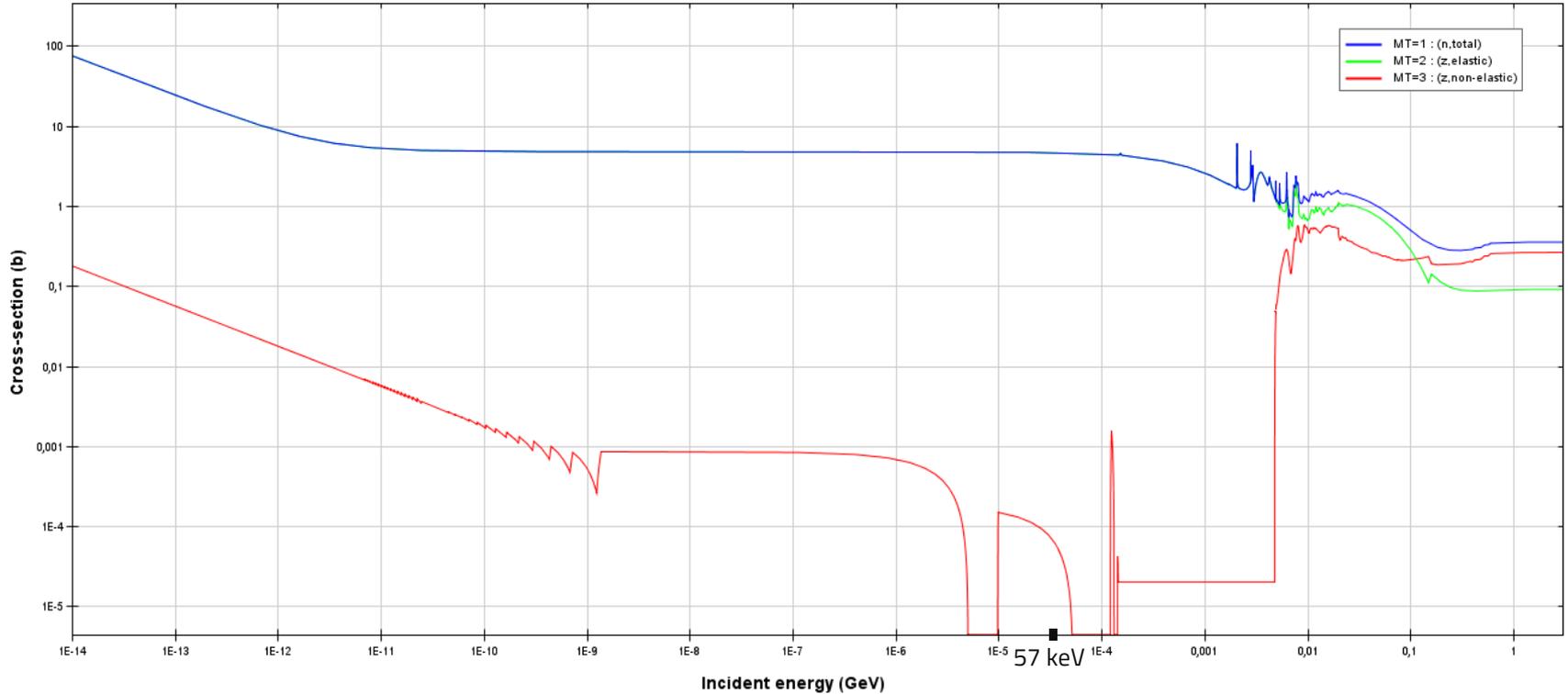
Incident neutron data // C13 //



ENERGY	EMISSION
[0.005; 0.05] GeV	<b><math>n + \text{C-13} \rightarrow 2n + \text{C-12}</math></b>
[0.01; 0.05] GeV	<b><math>n + \text{C-13} \rightarrow \alpha + \text{Be-10}</math></b>
[0.01; 0.1] eV + [1 × 10 <sup>-5</sup> ; 5 × 10 <sup>-4</sup> ] GeV	<b><math>n + \text{C-13} \rightarrow \gamma + \text{C-14}</math></b>
[0.01; 0.1] GeV	<b><math>n + \text{C-13} \rightarrow \text{H-3} + \text{B-11}</math></b>

# Total, Elastic and Non-Elastic $^{13}\text{C}$ Cross Section

Incident neutron data / JENDL/HE-2007 / C13 // Cross section



	PREDOMINANT CROSS-SECTION
ELASTIC	$[1 \times 10^{-14}; 0.1]$ GeV
NON-ELASTIC	$[0.1 ; 1]$ GeV